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# Parameter Uncertainty for Repository Thermal Analysis

**Fuel Cycle Research & Development** 

Prepared for: U.S. Department of Energy Used Fuel Disposition Campaign Ernest Hardin and Teklu Hadgu Sandia National Laboratories Harris Greenberg Lawrence Livermore National Laboratory Mark Dupont Savannah River National Laboratory April 2012 FCRD-UFD-2012-000097



### Parameter Uncertainty for Repository Thermal Analysis

Deliverable M4FT-12SN0804033 Work Package FT-12SN080403

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#### Summary

This report describes three contributions to analysis of uncertainty in maximum repository (peak waste package surface) temperatures:

- Analytical description of overall variance in temperature, as a function of contributions from key parameters of the analytical solution used in previous temperature analyses.
- Compilation of literature data on key parameter values, for various geologic host media and clay-based buffer materials, drawing on international work.
- Correlation between maximum repository temperature and waste package power at emplacement, which is strong even for different combinations of waste package size, SNF burnup, and fuel age.

Results from the analytical treatment of temperature variance include the following:

- Temperature at all times is relatively insensitive to heat capacitance (volumetric heat capacity) given the state of knowledge represented by the assigned parameter variance.
- Buffer thermal conductivity is an important parameter in early time (except for disposal concepts that have no buffer), but the ratio of buffer radius to waste package radius may be a more important parameter depending on how much it is allowed to vary in developing a disposal concept.
- Host rock thermal conductivity is the most important input parameter, especially at later time (e.g., greater than 10 years after emplacement) and where buffers are not used.

Compilation of literature data and selection of uncertainty ranges provide mean and  $\pm 1\sigma$  property values for temperature uncertainty analysis based on the analytical solution.

Finally, the study of maximum (peak waste package surface) temperature corroborates earlier findings from FEM simulations of the generic salt repository. Thus the waste package heat output can be used to predict maximum temperature, to a good approximation, for all host media, waste package sizes, SNF burnup, and decay storage duration cases considered.

# **Revision History**

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# Acronyms

BWR	Boiling-Water Reactor
DOE	U.S. Department of Energy
EBS EDZ	Engineered Barrier System Excavation Damage Zone
FEM FY	Finite Element Method Fiscal Year
GW-d	Gigawatt-Days
HLW	High Level (Radioactive) Waste
LWR	Light Water Reactor
MT MTHM	Metric Ton Metric Tons of Heavy Metal
OCRWM	Office of Civilian Radioactive Waste Management (DOE)
OCRWM PWR	Office of Civilian Radioactive Waste Management (DOE) Pressurized Water Reactor
PWR	Pressurized Water Reactor
PWR R&D SNF	Pressurized Water Reactor Research and Development Spent Nuclear Fuel
PWR R&D SNF SNL TAD	Pressurized Water Reactor Research and Development Spent Nuclear Fuel Sandia National Laboratories Transport/Aging/Disposal

## Parameter Uncertainty for Repository Thermal Analysis

#### **1. Introduction**

This report is one follow-on to a study of reference geologic disposal design concepts (Hardin et al. 2011a). Based on an analysis of maximum temperatures, that study concluded that certain disposal concepts would require extended decay storage prior to emplacement, or the use of small waste packages, or both. The study used nominal values for thermal properties of host geologic media and engineered materials, demonstrating the need for uncertainty analysis to support the conclusions. This report is a first step that identifies the input parameters of the maximum temperature calculation, surveys published data on measured values, uses an analytical approach to determine which parameters are most important, and performs an example sensitivity analysis. Using results from this first step, temperature calculations planned for FY12 can focus on only the important parameters, and can use the uncertainty ranges reported here.

The survey of published information on thermal properties of geologic media and engineered materials, is intended to be sufficient for use in generic calculations to evaluate the feasibility of reference disposal concepts. A full compendium of literature data is beyond the scope of this report. The term "uncertainty" is used here to represent both measurement uncertainty and spatial variability, or variability across host geologic units. For the most important parameters (e.g., buffer thermal conductivity) the extent of literature data surveyed samples these different forms of uncertainty and variability. Finally, this report is intended to be one chapter or section of a larger FY12 deliverable summarizing all the work on design concepts and thermal load management for geologic disposal (M3FT-12SN0804032, due 15Aug2012).

### 2. Analytical Sensitivity

The purpose of this analysis is to determine the relative contribution of variance in each key parameter to overall variance in temperature, as calculated with the analytical method from the FY11 report (Hardin et al. 2011a, Section 5). A classical analysis-of-variance approach is used for thermal conductivity, heat capacity, and buffer radius ratio parameters, while a direct approach is used for sensitivity to initial waste heat output. In the classical approach, variance in key parameters and in overall temperature, is evaluated for parameter values in the local vicinity of solutions used to represent each reference disposal concept (i.e., this is not a global variance analysis). This limitation is appropriate because the purpose of this analysis is to gain insight on generic disposal concepts, where each input parameter can be considered separately and the parameters are uncorrelated. For a particular site, other approaches such as Monte Carlo sampling could be used with more definitive parameter support (e.g., separately quantified spatial variability).

The direct approach is described in Section 5, and is used to show the correlation between maximum temperature and waste package power at emplacement (taking into account the form of the thermal decay function). Such correlations for SNF disposal in each host medium are developed for use in system studies that impose thermal power limits on SNF packaging, storage, transport, or disposal.

#### 2.1 Analytical Derivation of Temperature Sensitivity

A simplified model is used for uncertainty analysis, that represents both the transient and steadystate parts of the analytical model from the FY11 report. The transient part is represented using an analytical line-source solution (Hardin et al. 2011a, Section G.3) and the steady-state part is represented by adding the same axisymmetric function (Section G.4). This approach departs from the full solution documented in the FY11 report, only with respect to the difference between an infinite line source, and a finite line source plus an array of point sources. For this analysis heating from adjacent drifts is less important for peak near-field temperatures that occur in the first few years after waste emplacement (Hardin et al. 2011a; comparing the sum of the central package and adjacent package contributions, to the adjacent drift contributions in Figures 5.3-3 and 5.3-4). Whereas the model developed in the FY11 report used multiple annular layers to represent the EBS (Hardin et al. 2011a, Figures 5.1-1, and 5.1-3 through 5.1-6), the simplified model used here uses a single annular layer to address the impact of an uncertain thermal resistance (e.g., dominated by a clay buffer) on the total variance for temperature. The simplified model for temperature *T* as a function of radius *r* (with  $r = r_2$ ) and time *t*, to be used for parameter uncertainty analysis, has the form:

$$T(t) = T_{amb} + \frac{Q(t)}{2\pi K_{buf}} \ln\left(\frac{r_2}{r_1}\right) + \int_0^t \frac{Q(\tau)}{4\pi K_{rock}(t-\tau)} e^{\frac{-r_2^2 \rho C_p}{4K_{rock}(t-\tau)}} d\tau$$
 Eqn. 1

where	K <sub>rock</sub>	=	rock thermal conductivity (W/m-K)
	$\rho C_p$	=	rock heat capacitance (volumetric heat capacity; J/m <sup>3</sup> -K)
	K <sub>buf</sub>	=	buffer thermal conductivity (W/m-K)
	Q(t)	=	the line source strength (W/m)
	$r_1$	=	waste package radius (m)
	$r_2$	=	buffer radius (m)
	$T_{amb}$	=	the ambient (far-field) rock temperature (°C)

The line-source temperature at radius  $r_2$  (outer radius of the buffer, or alternatively, the interface between the EBS and the host rock) represented by the third term on the right-hand side of Equation 1, is increased by the second term to account for heat conductance across a simple single-layer annular buffer. Additional layers could be added to the engineered barriers between the waste package and the rock, such as a metallic liner or envelope, but metallic layers have little effect on peak waste package surface temperature because they are thin and have high thermal conductivity. The overall variance of T(t) is given by (Hahn and Shapiro (1967, p. 231):

$$Var\{T(t)\} = \sum_{i=1}^{n} \left(\frac{\partial T}{\partial z_i}\right)^2 Var\{z_i\}$$
 Eqn. 2

where there are *n* parameters  $z_i$ . Analytical expressions for  $\frac{\partial T}{\partial z_i}$  were symbolically calculated

using MathCad14<sup>®</sup>, with respect to parameters  $K_{rock}$ ,  $K_{buf}$ ,  $\rho C_p$ , and  $r_2/r_1$ . The calculation point for maximum temperature is selected as the waste package wall, as it was for the FY11 analysis, because package contents can withstand higher temperatures than the ex-container engineered

materials, particularly clay buffers. Spent fuel zirconium alloy cladding can withstand 350°C for indefinite (postclosure) duration (BSC 2008a, Section 11.2). Following the methodology outlined by BSC (2008b, Section 6.1.6) for 21-PWR transport-aging-disposal (TAD) canisters, fuel cladding will not exceed 350°C even for waste package surface temperatures up to approximately 300°C. The highest waste package surface temperature considered in this body of work is 250°C as discussed for the salt repository (Hardin et al. 2011a, Section 4.1.1.2).

Materials outside the waste package (clay-based buffer, intact clay/shale, crushed salt backfill, intact salt, etc.) have maximum allowable temperatures (e.g., 100°C or possibly higher for clay buffers, 200°C for crushed salt; see Hardin et al. 2011a) that are less than the limit on waste package wall temperature associated with package contents. These materials are therefore limiting for management of waste heat after emplacement.

Framing parameter uncertainty as a variance analysis for this model, incorporates not only the

functional dependence  $\frac{\partial T}{\partial z_i}$  but also the range of variability for key parameters, expressed in

 $Var{z_i}$ . Values for  $Var{z_i}$  are estimated in the next section using ranges reported in the literature for similar materials.

With the addition of open modes to the portfolio of reference disposal options, thermal analysis must account for the heat removed by preclosure ventilation, and radiative heat transfer across gaps whenever ventilation is not effective. During ventilation, heat removal decreases the power dissipated to the waste package surroundings by as much as a factor of 6 (BSC 2004) so there is no possibility of exceeding near-field temperature limits for normal operation. (Off-normal operations are beyond the scope of this report.) For heat transfer across gaps, the effect is similar to a high effective thermal conductivity (r.g., see BSC 2004, Equation 6-53).

Selection of the ratio  $r_2/r_1$  is based on the idea that buffer size  $(r_2)$  would be selected after waste package size  $(r_1)$ . For the four key parameters  $K_{rock}$ ,  $K_{buf}$ ,  $\rho C_p$ , and  $r_2/r_1$  of the model (Equation 1) the derivatives comprising the right-hand side of Equation 2 are represented by:

$$\frac{d}{dKrock}T(t) = \int_{0}^{t} \frac{\frac{\rho C p \cdot \left(r_{2}\right)^{2}}{e^{\frac{1}{4} \cdot Krock \cdot (\tau - t)} \cdot Q(\tau)}}{4 \cdot \pi \cdot Krock^{2} \cdot (\tau - t)} + \frac{\rho C p \cdot \left(r_{2}\right)^{2} \cdot e^{\frac{\rho C p \cdot \left(r_{2}\right)^{2}}{4 \cdot Krock \cdot (\tau - t)} \cdot Q(\tau)}}{16 \cdot \pi \cdot Krock^{3} \cdot (\tau - t)^{2}} d\tau$$

Eqn. 3

$$\frac{d}{dKbuff}T(t) = \cdot -\frac{\ln\left(\frac{r_2}{r_1}\right) \cdot Q(\tau)}{2 \cdot \pi \cdot Kbuff^2}$$
Eqn. 4

$$\frac{d}{d\rho Cp}T(t) = \int_{0}^{t} \frac{\rho Cp \cdot (r_2)^2}{-(r_2)^2 \cdot e^{\frac{\rho Cp \cdot (r_2)^2}{4 \cdot Krock \cdot (\tau - t)} \cdot Q(\tau)}} d\tau$$
Eqn. 5

$$\frac{d}{dr2r1}T(t) = \cdot \frac{Q(\tau)}{2 \cdot \pi \cdot Kbuf \cdot r2r1} + \int_{0}^{t} \frac{\left(r_{2}\right)^{2} \cdot \rho Cp \cdot e^{\frac{\left(r_{2}\right)^{2} \cdot \rho Cp}{4 \cdot Krock \cdot (\tau - t)} \cdot Q(\tau)}}{8 \cdot \pi \cdot Krock^{2} \cdot (\tau - t)^{2}} d\tau$$
Eqn. 6

where parameter r2r1 represents  $r_2/r_1$ . Note that the derivative expressions above are presented in MathCad® syntax as total derivatives, whereas Equation 2 is written with partials. The distinction is not meaningful here because the parameters of interest as applied here are essentially independent, uniform (within the domains where they apply), and not time-varying (e.g.,  $K_{buf}$  is an effective value). Thus, the two types of derivatives are equivalent for this analysis.

Each of the expressions above (Equations 3 through 6) contains an integral which is the convolution of the thermal decay function and must be evaluated numerically. The results from applying these derivatives are presented below as the unnormalized magnitudes of the differentials  $\frac{\partial T}{\partial z}$  (Figure 1). The differentials with respect to thermal conductivities of the rock

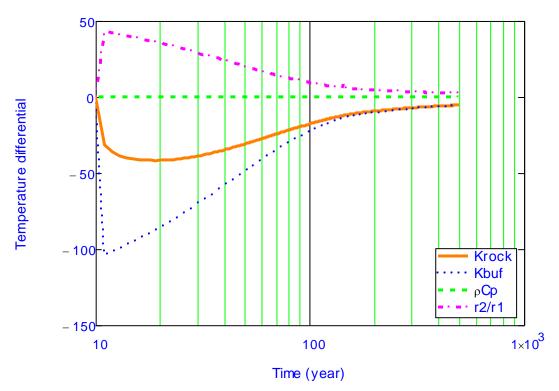
and buffer are negative, as expected, while the differential with respect to  $r_2/r_1$  is positive, and that with respect to  $\rho C_p$  is close to zero. Because these curves are unnormalized and therefore have different dimensions, they cannot be compared directly in magnitude. The variance approach outlined above is used in Section 4 below to facilitate direct comparison.

The above discussion does not consider uncertainty on the heat input, i.e., the line source strength Q(t). Uncertainty in heat output from SNF or HLW is related to uncertainty in composition, particularly the major heat-producing fission products (Cs-137 and Sr-90) and actinides such as Am-241. Whereas uncertainty in heat output of SNF assemblies is possible, it is

not treated as parametric uncertainty here because it is relatively small compared to that associated with other parameters such as clay buffer characteristics and host rock thermal conductivity. Also, the uncertainty for a waste package containing multiple assemblies decreases statistically, to the extent that the variability among assemblies is uncorrelated. The potential effects of uncertainty in waste package heat output on maximum temperature can be readily visualized using the correlations developed in Section 5.2.

#### **3.** Parameter Uncertainty Ranges

This section presents a limited survey of literature data for Krock and  $\rho$ Cp, specific to crystalline rock, clay/shale media, salt, and alluvium. It also reviews literature data for Kbuf, for clay buffer materials and other engineered materials. The important result of these reviews is a set of low-high ranges for each parameter, for each host medium. Treating the reported literature data as samples from populations of independent data, the sample mean and sample standard deviation were calculated (Tables 1 though 4).



Note: For the case of crystalline host rock, 4-PWR waste packages (0.66 m diameter), 0.35 m buffer thickness, and SNF with 40 GW-d/MT burnup (10 yr out-of-reactor). For buffer thermal conductivity the average of dry and hydrated values was used.

Figure 1. Unnormalized (dimensional) Partial Derivatives of Temperature at the Waste Package Surface with Respect to Key Model Parameters ( $K_{rock}$ ,  $K_{buf}$ ,  $\rho C_p$ , and  $r_2/r_1$ ), for the Crystalline Rock SNF Disposal Reference Case.

The estimated sample statistics are useful to describe the uncertainty in these key parameters, subject to limitations because the literature data were not all produced the same way, have associated measurement errors or biases, and in some instances the data are sparse. Accordingly, a range selection is also provided, rounding the estimates (mean  $\pm$  standard deviation) up or down, consistent with qualitative indications of the reproducibility and stability in reported data of each type. The adjusted estimate of the standard deviation is then half the selected range, and the estimated parameter variance for use in this study is the square of the standard deviation.

# 3.1 Host Rock Thermal Conductivity

Thermal conductivity for the different host geologic media are shown in Table 1, derived from a collection of literature (much of which is related to geologic disposal of heat-generating waste). Some of the sources have provided ranges, and the endpoints are treated as separate estimates thereby assigning twice the weight to these sources.

Granite and other crystalline rocks (metamorphic or igneous) have small porosity, and thermal conductivity is not sensitive to the state of moisture saturation. Hence, the values used here do not distinguish saturation state. The calculated standard deviation of  $K_{rock}$  is 0.37, but the selected range (representing  $\pm 1\sigma$ ) has a width of 0.8 allowing for some unknown variations in porosity, saturation, measurement method, spatial variability, etc. The variance  $Var\{K_{rock}\}$  is estimated from the square of the estimated standard deviation (0.37<sup>2</sup> = 0.14, units of (W/m-K)<sup>2</sup>).

For clay and shale media the range is broader reflecting the incorporation of indurated shales and non-indurated plastic clays. Also, the parallel and perpendicular orientations are lumped together, which is appropriate for a temperature solution for isotropic media (the estimates could be split for anistropy calculations). For in situ or intact measurements on these materials, the moisture content is assumed to be close to undisturbed conditions, so the values here do not distinguish saturation state. The excavation damage zone (EDZ) measurement of Johnson et al. (2002) is included, but falls near the middle of the selected range.

For salt two thermal conductivity ranges are selected, for 100°C and 200°C. Only one value for 200°C is presented here (model based, supported by experimental data) so the standard deviation from the 100°C data is used for the 200°C case. Salt has low porosity and moisture content so thermal conductivity is not sensitive to the state of moisture saturation.

Alluvium has high porosity (on the order of 30% or greater) and volumetric (total) moisture content from approximately 5% to 20% (i.e., moisture saturation up to 70% or greater). Accordingly, two ranges are presented for in situ or unsaturated conditions, and for wet or saturated conditions. Only data for naturally consolidated (not re-consolidated) samples or in situ measurements are presented. The unsaturated data are recommended for use with the unsaturated, sedimentary disposal concept, although the "wet" or saturated data could be used with justification.

Finally, no literature data survey is provided for the crystalline basement host medium. This is a somewhat generic category of rock types so a wide range of igneous and metamorphic rock types is possible. Importantly, the FY11 analysis (Hardin et al. 2011a, Section 5) showed that maximum temperatures for the deep borehole concept would be relatively low because of the small diameter and limited waste content of the canisters. Further, no temperature limits were

identified, so the uncertainty in basement rock thermal conductivity does not appear to be a significant factor, and no analysis is provided here.

## 3.2 Engineered Material Thermal Conductivity

Thermal conductivity for clay buffer materials described in the geologic disposal literature, and for metals and alloys used in waste packages, are presented in Table 2. Thermal properties of dry and hydrated bentonite clay-based buffer materials are well studied. Dry and hydrated data are separated in Table 2, and the modeling strategy should determine the state of the buffer for which maximum temperatures are calculated. An intermediate value was used in previous analysis (Hardin et al. 2011a, Section 5.3.2) subject to verification by modeling or experiment.

The results for metals and alloys show that the range of uncertainty is small for all materials except stainless steels, which exhibit variation with differences in type and composition. Regardless, thermal conductivities for all these materials are great enough, and thickness small enough, that they have no significant impact on maximum temperatures (even if used in the engineered barrier system outside of the waste package, such as for liners).

# 3.3 Host Rock Heat Capacity

Heat capacitance (volumetric heat capacity) for the different host geologic media are shown in Table 3. A somewhat different approach was used for range selection, to allow for additional uncertainty due to saturation and compositional differences, and more direct comparison of the different media. The ranges use the estimated sample mean as the midpoint, and are guided by the calculated standard deviations. For low porosity salt and granite, a  $\pm 5\%$  range is selected, while for clay/shale and alluvium, ranges of  $\pm 15\%$  and  $\pm 20\%$  are selected, respectively. For the crystalline basement (deep borehole) a range of  $\pm 10\%$  is assumed.

# 3.4 Waste Package and Buffer Size

The buffer size ratio parameter  $r_2/r_1$  is an engineering detail and not a material physical property. However, it is shown by this report to be an important parameter in the uncertainty of maximum temperature predictions, and a "variance" is estimated to reflect the need for flexibility in the engineering details of disposal concepts. As shown in Table 4, the variance is approximated using a ±50% range about a nominal buffer thickness  $(r_2 - r_1)$ , for a range of waste package sizes  $(r_1)$ .

The open-mode data in Table 4 are suitable for use in calculations that involve pre-closure ventilation (reducing Q(t)) followed by cessation of forced ventilation at or before repository closure, and either: 1) installation of a backfill around the waste packages, or 2) leaving the air space open around the waste packages. In the first instance, the temperatures after backfilling can be calculated using a model that includes the backfill even during preclosure ventilation, but adjusts Q(t) to account for ventilation heat removal (this is consistent with the model in Equation 1 because the annular EBS term is steady-state). The point of temperature calculation may be chosen at the waste package surface, to limit the maximum temperature of the engineered backfill. In the second instance, the backfill is replaced by an effective thermal conductivity for the air space throughout the calculation. This is a good approximation because the thermal resistance of an air gap is much lower than backfill or buffer material (effective  $K_{th}$  much greater

than buffer conductivity  $K_{buf}$ ). In this second instance the point of temperature calculation may be selected at the rock wall or the waste package depending on which is limiting.

#### 3.5 Summary

Key parameters ( $K_{rock}$ ,  $K_{buf}$ ,  $\rho C_p$ , and  $r_2/r_1$ ) were identified for an analytical solution for repository temperatures, that represents the analysis approach used in the FY11 report (Hardin et al. 2011a). Literature data were compiled (Tables 1 through 3) and uncertainty ranges selected for different host geologic media and for clay-based buffers. The buffer radius ratio (expressed as  $r_2/r_1$ ) was similarly described using ±50% variation around the reference values (Table 4; reference values from Hardin et al. 2011a). The results consisting of average and low, high (±1 $\sigma$ ) values for each parameter, are intended for use in temperature uncertainty analyses using calculation approaches based on Equation 1.

Parameter ranges for  $K_{rock}$ ,  $K_{buf}$ ,  $\rho C_p$ , and  $r_2/r_1$  from Section 2 are converted to estimates of variance representing for host media, and clay-based buffers, using the range endpoints as estimates of  $\pm 1\sigma$  values. The sample variance is then estimated from

$$Var\{z_i\} \approx \left[\frac{\left(Range_{high} - Range_{low}\right)_i}{2}\right]^2$$
 Eqn. 7

The resulting sample variance estimates for the key parameters  $K_{rock}$ ,  $K_{buf}$ ,  $\rho C_p$ , and  $r_2/r_1$ , along with the nominal values of these parameters for reference disposal concepts, are summarized in Table 5.

Host Rock	Therr	nal Conductivity															
Low	High	Source	Average	Std. Dev.													
Granite																	
2.50		Andra 2005a	2.81	0.37	-							<ul> <li>Andra</li> </ul>	a 2005a				
2.77		SKB 2006 (Laxemar)			0.5	1.0	1.5	2.0	2.5	3.0	25		006 (Laxem				
3.34		SKB 2006 (Forsmark)			0.5	1.0				3.0	3.5		006 (Forsma na and Hellä	,	1		
2.61		Pastina and Hellä 2010 (60°C)			-		Therma	al Conductiv	ity (W/m-K)				e Selection	2010 (00 C	1		
2.4		Range Selection										-					
Clay/Shale																	
1.75		Jia et al. 2009	1.73	0.61													
1.70		ONDRAF/NIRAS 2001 (Boom clay)														+	
0.70		ONDRAF/NIRAS 2001 (Ypresian clay)			-							🔶 Jia et i					
1.3		Andra 2005b (perpendicular)			-								AF/NIRAS 20 AF/NIRAS 20				
1.9		Andra 2005b (parallel)			-								2005b (per				
1.8		Johnson et al. 2002 (Upper Opalinus, perp.)			<b>-</b>		**	X+₩X	+			+ Andra	2005b (para	allel)			
3.2		Johnson et al. 2002 (Upper Opalinus, parallel)			0.5	1.0	1.5	2.0	2.5	3.0	3.5				palinus, perp palinus, para		
1.3		Johnson et al. 2002 (Lower Opalinus, perp.)			-		_								palinus, para		
2.0		Johnson et al. 2002 (Lower Opalinus, parallel)			-		Therma	al Conductiv	ity (W/m-K)						palinus, paral	ilel)	
1.5		Johnson et al. 2002 (Opalinus, EDZ)			-								on et al. 200 & Marivoet		, EDZ)		
1.35		Sillen & Marivoet 2007			-								Selection	2007			
1.1		Range Selection															
Salt																	
5.4		Clayton & Gable 2009 (27°C)	4.88	0.53	-							<ul> <li>Clavto</li> </ul>	n & Gable 2	009 (27°C)			
4.2		Clayton & Gable 2009 (100°C)			1				×		<b></b>		n & Gable 2		)		
4.7		Fluor 1985 (110°C)			2.5	3	3.5	4	4.5	5	5.5		n & Gable 2		)		
5.2		Fluor 1986 (47°C)											1985 (110°C	.)			
3.2		Clayton & Gable 2009 (200°C)	3.21	0.53			Therma	l Conductiv	ity (W/m-K)			+ Fluor 1986 (47°C) Range Selection (200°C)					
4.4	5.4	Range Selection (100°C)										0	Selection (1	,			
2.7		Range Selection (200°C)												,			
Alluvium																	
1.05		Wollenberg et al. 1982 (in situ)	1.06	0.11													
0.91		Wollenberg et al. 1983 (downhole probe)										<ul> <li>Wolle</li> </ul>	nberg et al.	1982 (in situ	u)		
1.0		Smyth et al. 1979 (unsat., consolidated)										▲ Wolle	nberg et al. :	1982 (satura	, ated, consolio		
1.0		Range Selection (unsat., consolidated)			- 		X								hole probe)		
0.98		Wollenberg et al. 1982 (wet, consolidated)	1.49	0.34	0.0	0.5	1.0	1.5	2.0	2.5	3.0				consolidated)		
1.21	1.81	Wollenberg et al. 1982 (wet, consolidated)					Therma	al Conductiv	ity (W/m-K)						consolidated)		
1.51		Wollenberg et al. 1982 (wet, consolidated)					merina					<ul> <li>Smyth</li> </ul>	et al. 1979	(unsat., con	solidated)		
1.5		Wollenberg et al. 1982 (saturated, consolidated)										Range	Selection (u	insat., conse	olidated)		
1.2		Range Selection (saturated, consolidated)														T	
Crystalline																	
3.0		Brady et al. 2009			No tempe	rature lim	its identifi	ied for dee	p borehole (	disnosal	concent (H	ardin et al	2011)		_		

Table 1. Host Rock Thermal	Conductivity Range	s and Parameter Variance

Engine	ered N	laterial Thermal Conductivity															
Low	High	Source	Average	Std. Dev.													
Clay Bu	uffer (d	ompacted)											🔷 Johr	nson et al. 20	01 (2% moi	sture)	
0.4	+	Johnson et al. 2001 (2% moisture)	0.42	0.05										/ 1993 (comp			
0.39	)	Gray 1993 (compacted, dry)			0.0		0.5	1.0	1.5	2.0	2.5	3.0		ra 1985 (2%) kaert et al. 1		am Claul	
0.5	5	Nagra 1985 (2% moisture)			0.0		0.5				2.5	5.0		ra 1985 (2%)		Uni Clay)	
0.4	L .	Volckaert et al. 1996 (dry Boom Clay)						Thermal C	Conductivity	(W/m-K)				RAF/NIRAS			
0.3	0.	Range Selection (dry)												ge Selection			
1.35	i	Nagra 1985 (2% moisture)	1.43	0.11									Ran	ge Selection	(hydrated)		
1.5	i	ONDRAF/NIRAS 2001															
1.3	1.	Range Selection (hydrated)				1						1					L
Stainle	ess Ste	el									· · · · · · · · · · · · · · · · · · ·	<b>A</b> -1		etjens and S			
17.0	)	Weetjens and Sillen 2005 (stainless)	16.7	2.26	0		5		10	1	.5	20		nsenow et al. aith 1965 (SS			
14.4	1	Rohsenow et al. 1985 (SS316 at 737°C)						Thermal	Conductivity	(W/m-K)				nge Selection			
18.9	)	Kreith 1965 (SS304 at 300 C)															
14.4	1	Range Selection (all stainless)															
Carbon	Steel																
50.0	)	Andra 2005b	49.0	3.61		-	10		25 20	25 40				dra 2005b Inson et al. 2	002		
52.0	)	Johnson et al. 2002			0	5	10 1	15 20 Thermal	25 30 Conductivity	35 40	45 50	55		or 1985	002		
45.0	)	Fluor 1985						merma	conductivity	(00711110)			• 114	01 1505			
45.4	52.	Range Selection (carbon steel)															
Coppei	r																
380.9	)	Rohsenow et al. 1985 (300°C)	378.6	5 10.73			I I	1	1					isenow et al.		C)	
366.9	)	Kreith 1965 (300°C)			0	5	0 100	0 150	200	250 30	0 350	400		ith 1965 (300 ast 1968 (22			
388.0	)	Weast 1968 (227°c)						Thermal C	onductivity	(W/m-K)				ige Selection			
367.9	389.	Range Selection (copper)															
Crushe	d Salt	(partially consolidated)															
0.46	5	Fluor 1985	0.47	0.01									Elui	or 1985			
0.47	'	Bechtold et al. 2004 (30% porosity, 100°C)								<b></b>					004 (30% p	orosity, 100°	c)
0.4	0.0	Range Selection (30% porosity, 100°C)			0.0		0.5	5	1.0	1.	5	2.0				orosity, 200°	
1.34	L	Bechtold et al. 2004 (20% porosity, 200°C)	1.34	0.01				Thermal (	Conductivity	(W/m-K)			Ran	nge Selection	(30% poros	ity, 100°C)	
1.2	1.4	Range Selection (20% porosity, 200°C)											Rar	nge Selection	(20% poros	ity, 200°C)	

Table 2. Engineered Material Thermal Conductivity Ranges and Parameter Variance

HOST KOC	k Heat Ca	pacitance																
Gravim.	Dry Bulk	Volum.						_										
Heat Cap.	Density	Heat Cap.			Std.													
	(kg/m^3)	(J/m^3-K)		Avg.	Dev.													
Salt						_												
931	2190	2.04E+06	Clayton and Gable 2009	2.02E+06	2.88E+04								٠	Clayton and	Gable 2009			
920	2162	1.99E+06	Fluor 1986			1.0E+0	6	1.5E+06	2.0E+0	16	2.5E+06	3.0E+06		Fluor 1986				
931	2190	2.04E+06	Fluor 1985			1.0210	0		etric Heat C			3.0L100		Fluor 1985				
			Range Selection (±5%): 1.	92E+06 to 2	.12E+06								_	Range Selec	tion (±5%)			
Granite																		
837.5	2650	2.22E+06	Andra 2005a	2.22E+06	5.01E+04								•	Andra 2005				
837.5	2700	2.26E+06	SKB 2006 (Laxemar)			1.0E+0	6	1.5E+06 2.0E+06 2.5E+06				3.0E+06		<ul> <li>SKB 2006 (Laxemar)</li> <li>SKB 2006 (Forsmark)</li> </ul>				
837.5	2700	2.26E+06	SKB 2006 (Forsmark)			1.0E+0	0		2.0E+0 etric Heat C			3.UE+Ub	- <b>x</b>		Hella 2010 ((	@60C)		
784	2749	2.16E+06	Pastina and Hella 2010 (@60C)					volun		apacity (J/	··· -Nj	)		Range Selection (±5%)				
			Range Selection (±5%): 2.	11E+06 to 2	.34E+06													
Clay/Shal	e																	
1005	2700	2.71E+06	Jia et al. 2009	2.51E+06	2.92E+05								•	Jia et al. 200	19			
	2400	2.30E+06	Johnson et al. 2002 (Opalinus Clay)			1.0E+0	6	1.5E+06	2.0E+0		2.5E+06	3.0E+06						
			Range Selection (±15%):	2.13E+06 to	2.88E+06	1.01+0	0		etric Heat C			3.02+00	-	<ul> <li>Johnson et al. 2002 (Opalinus Clay)</li> <li>Range Selection (±15%)</li> </ul>				
Deep Bor	ehole (Cr	ystalline B	asement)			-L												
790	2750	•	Brady et al. 2009	2.17E+06	NA													
			Range Selection (±10%):	1.96E+06 to	2.39E+06		r					1	٠	Brady et al.	2009			
Alluvium						1.0E+0	6	1.5E+06 Volum	2.0E+0 etric Heat C		2.5E+06 m <sup>3</sup> -K)	3.0E+06	_	Range Selec	tion (±10%)			
1000	1700	1.70E+06	Smyth et al. 1979	1.46E+06	2.31E+05			• orun	et. le lieut e	apacity (J)								
1000	1200	1.20E+06	Smyth et al. 1979															
836	1600	1.34E+06	Wollenberg et al. 1983															
1000	1600	1.60E+06	Wollenberg et al. 1983				-						•					
			Range Selection (±20%):	1.17E+06 to	01.75E+06	1.0E+0	6	1.5E+06 Volum	2.0E+0 etric Heat C		2.5E+06 m <sup>3</sup> -K)	3.0E+06	×		et al. 1983			
															(±20/0)			

Table 3. Host Rock Heat Capacitance (Volumetric Heat Capacity) Ranges and Parameter Variance

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Buffer:Wast	e Package	Radius Ratio						
			Thickness Ran	ge Selection	Ratio Range	e Selection	Std. Dev.	Variance
Waste Package	Radius (m)	Buffer Thickness (m)		+50%	r2/r1 (-50%)	r2/r1 (+50%)	r2/r1	r2/21
Crystalline R	lock (Clay l	buffer)						
1-PWR	0.17	0.35	0.175	0.525	2.06	4.18	1.06	1.12
4-PWR	0.33	0.35	0.175	0.525	1.53	2.59	0.53	0.28
1-HLW	0.30	0.35	0.175	0.525	1.57	2.72	0.57	0.33
Clay/Shale (	Enclosed N	/lode)						
1-PWR	0.17	0.7	0.35	1.05	3.12	7.36	2.12	4.50
4-PWR	0.33	0.7	0.35	1.05	2.06	4.18	1.06	1.12
1-HLW - no buff	er							
Salt (Enclose	ed Mode) -	no buffer						
Deep Boreh	ole - neglig	gible buffer thern	nal resistanc	e				
Shale Open	Mode (bac	kfilled or open at	t closure)					
4-PWR	0.33	1.82	0.91	2.73	3.76	9.27	2.76	7.60
12-PWR	0.62	1.53	0.765	2.295	2.23	4.70	1.23	1.52
21-PWR	0.90	1.25	0.625	1.875	1.69	3.08	0.69	0.48
32-PWR	1.00	1.15	0.575	1.725	1.58	2.73	0.58	0.33
Sedimentary	/ Open Mo	de (alluvium, bad	ckfilled at clo	sure)				
4-PWR	0.33			2.73	3.76	9.27	2.76	7.60
12-PWR	0.62	1.53	0.765	2.295	2.23	4.70	1.23	1.52
21-PWR	0.90	1.25	0.625	1.875	1.69	3.08	0.69	0.48
32-PWR	1.00	1.15	0.575	1.725	1.58	2.73	0.58	0.33

Table 4. Buffer: Waste Package Radius Ratio Ranges and Parameter Variance

### 4. Variance Estimates for Temperature

Variance estimates from Table 5 are then used with Equation 2, to calculate contributions to the overall temperature variance from the variance assigned to each parameter. Figures 2 through 4 contributions from key parameters, with and without a buffer. For Figure 3 the temperature is normalized to temperature using Equations 1 and 2:

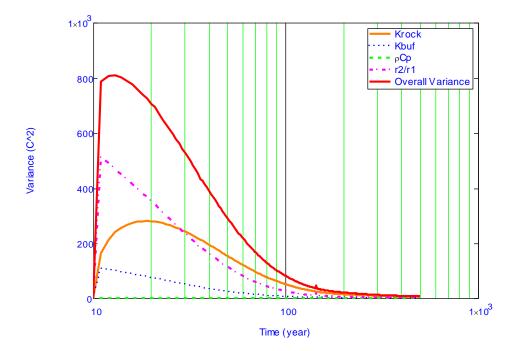
Normalized temperature variance = 
$$\frac{Var\{T(t)\}}{T^2(t)}$$
 Eqn. 8

The discussion below focuses on the un-normalized, time-varying temperature variance (Figures 2 and 4) which applies directly to temperature histories calculated for these cases. (The standard deviation of temperature uncertainty can be estimated by taking the square root of variance in Figures 2 and 4.)

Figure 2 summarizes the results of the analytical sensitivity analysis. The components of variance are summed to generate the overall variance on temperature. The  $K_{buf}$  (buffer) curve can be neglected and the overall variance adjusted accordingly, for concepts with no buffer (e.g., Figure 4). Similarly, the  $r_2/r_1$  curve can be ignored for applications where the waste package and buffer diameters are known.

Medium	Range Selection [Low High]	Nominal Value	Std. Deviation	Variance						
Host Rock Thermal Conductivity (W/m-K)										
Granite	[2.4 3.2]	2.8	0.37	0.16						
Clay/Shale	[1.1 2.3]	1.7	0.61	0.36						
Salt (100°C)	[4.4 5.4]	4.9	0.53	0.25						
Salt (200°C)	[2.7 3.7]	3.2	0.53	0.25						
Alluvium (unsaturated)	[1.0 1.2]	1.1	0.11	0.01						
Alluvium (saturated)	[1.2 1.8]	1.5	0.34	0.09						
Crystalline Basement	NA	3.0	NA	NA						
Engineer	ed Material Thermal	Conductivity	(W/m-K)							
Clay Buffer (dry)	[0.3 0.5]	0.4	0.05	0.01						
Clay Buffer (hydrated)	[1.3 1.5]	1.4	0.11	0.01						
Stainless Steel	[14.4 19]	16.7	2.26	5.3						
Carbon Steel	[45.4 52.6]	49.0	3.6	13.0						
Copper	[367.9 389.3]	378.6	10.7	114.5						
Crushed Salt (100°C)	[0.4 0.6]	0.5	0.01	0.01						
Crushed salt (200°C)	[1.2 1.4]	1.3	0.01	0.01						
Ho	st Rock Heat Capac	itance (J/m3-	K)							
Granite	[2.11E6 2.34E6]	2.23E6	5.0E4	3.3E9						
Clay/Shale	[2.13E6 2.88E6]	2.5E6	2.9E5	1.4E11						
Salt	[1.92E6 2.12E6]	2.0E6	2.9E4	1.0E10						
Alluvium	[1.17E6 1.75E6]	1.46E6	2.3E5	8.4E10						
Crystalline Basement	[1.96E6 2.39E6]	1.18E6	NA	4.6E10						
В	uffer:Waste Packag	e Radius Rati	0							
Granite (enclosed 4-PWR)	[1.53 2.59]	2.06	0.53	0.28						
Clay/Shale (enclosed 4-PWR)	[2.06 4.18]	3.12	1.06	1.12						
Salt (enclosed all packages)		no buffe	er							
Alluvium (enclosed 21-PWR)	[1.69 3.08]	2.39	0.69	0.48						
Crystalline Basement (enclosed 1-PWR)										

Table 5. Parameter Variance Values Used in Analysis



Note: For the case of crystalline host rock, 4-PWR waste packages (0.66 m diameter), 0.35 m buffer thickness, and SNF with 40 GW-d/MT burnup (10 yr out-of-reactor). For buffer thermal conductivity the average of dry and hydrated values was used. Units of y-axis are (°C)<sup>2</sup>.

Figure 2. Contributions to Overall Un-normalized Variance of Temperature (Equation 2) at the Waste Package Surface, from Parameters ( $K_{rock}$ ,  $K_{buf}$ ,  $\rho C_p$ , and  $r_2/r_1$ ) for the Crystalline Rock SNF Disposal Reference Case from Figure 1.

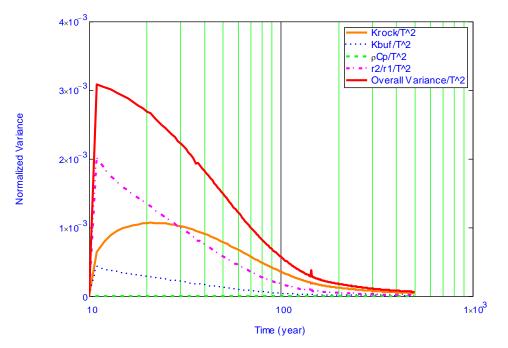
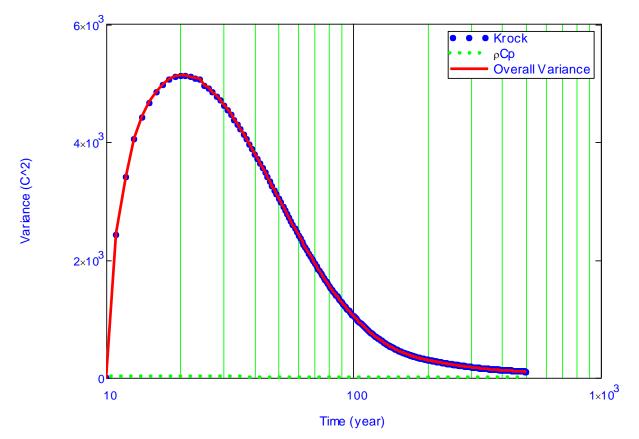


Figure 3. Normalized Variance of Temperature (Equation 8) at the Waste Package Surface, for the Crystalline Rock SNF Disposal Reference Case from Figures 1 and 2.

Uncertainty in heat capacitance  $(\rho C_p)$  has little or no influence on overall temperature uncertainty at all times (Figures 2 through 4) which is expected and consistent with the limited effect and narrow uncertainty ranges for this parameter. Uncertainty in buffer thermal conductivity is less important than the uncertainty in the buffer radius ratio parameter, given the uncertainty ranges assigned to each parameter (Table 5). Uncertainty in rock thermal conductivity ( $K_{rock}$ ) becomes most important (Figures 2 and 4) after an initial heating period. Similar figures can be generated for other disposal concepts using different waste types and package sizes, but the results are similar to those presented here.

A clay buffer can be a large thermal resistance (combining the effects of  $K_{buf}$  and  $r_2/r_1$ ), and potentially dominate the maximum temperature as shown by the unnormalized derivatives (Figure 1). Note that the partial derivatives are squared in Equation 2, so that differences in sign, e.g., between  $\partial T/\partial K_{buf}$  and  $\partial T/\partial (r_2/r_1)$ , do not appear in the overall variance (Figure 2). Only when the state of knowledge about  $K_{buf}$  as represented by  $Var\{K_{buf}\}$  is incorporated, is the variance contribution less than for the host rock ( $K_{rock}$ ) or buffer radius ratio ( $r_2/r_1$ ).



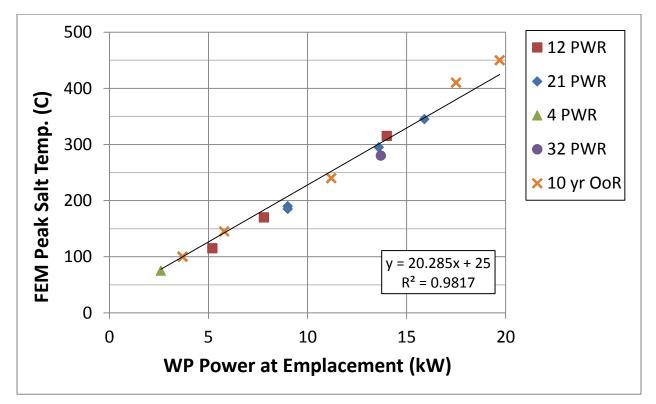
Note: For the case of salt host rock, 21-PWR waste packages (1.80 m diameter), and SNF with 40 GW-d/MT burnup (10 yr out-of-reactor). Thermal conductivity of intact salt at 200 °C was used. The concept does not involve a buffer, so this solution is based on a simple line-source calculation. Units of y-axis are (°C)<sup>2</sup>.

Figure 4. Contributions to Overall Un-normalized Variance of Temperature (Equation 2) at the Waste Package Surface, from Parameters ( $K_{rock}$  and  $\rho C_p$ ).

## 5. Maximum Temperature Sensitivity

## 5.1 Finite Element Based Correlation for Generic Salt Repository

An earlier study using finite-element (FEM) simulation of the generic salt repository (Clayton et al., 2012; Hardin et al. 2011b) showed that maximum salt temperature (peak temperature at the waste package surface) is correlated with the initial thermal power of the package at emplacement. The correlation applies over a wide range of package sizes, for a range of SNF burnup (Figure 5). This result is potentially useful as a thermal-power acceptance criterion for when SNF can be emplaced in a repository, in fuel management system studies. The correlation is further explored in the following section using the analytical solution (Equation 1) for different geologic host media, waste package sizes, SNF burnup, and decay storage periods.



Note: Calculations combine waste package sizes from Hardin et al. (2011a), SNF inventory from Carter and Luptak (2009), with the generic salt disposal concept (Carter et al. 2011), in a series of thermal and thermal-mechanical coupled calculations.

Figure 5. Correlation of Maximum Salt Temperature (Peak Package Surface Temperature) from a Set of Finite Element Simulations of the Generic Salt Repository (Clayton et al. 2012; Hardin et al. 2011a).

### **5.2 Analytical Line Source Correlations**

To corroborate the FEM results for salt, maximum temperatures are calculated for the FY11 reference disposal concepts for SNF using Equation 1, implemented in MathCad14®. For the

salt and deep borehole cases, no buffer is included (i.e., the second term on the right-hand side of Equation 1 was zero). Input parameters for these calculations include:

- Geologic host media: crystalline (granite), clay/shale, salt, crystalline basement (deep borehole)
- Waste package sizes: 4-, 12- and 21-PWR packages (0.33, 0.6 and 0.9 m diameter; Hardin et al. 2011a)
- Decay heat based on 40 and 60 GW-d/MTHM burnup (Carter and Luptak 2009)
- Surface storage times: 10, 20, 50, 100 years

The calculations are based on a surface temperature of 15°C, a geothermal gradient of 25°C/km, and a depth of 500 m (giving in situ temperature for the disposal depths described by Hardin et al. 2011a) except for the deep borehole calculation which is for a depth of 4 km. For this analysis only PWR assemblies (UOX) are considered, and waste package length is 5 m. Figure 6 shows decay heat for single SNF assemblies with burnup of 40 and 60 GW-d/MTHM.

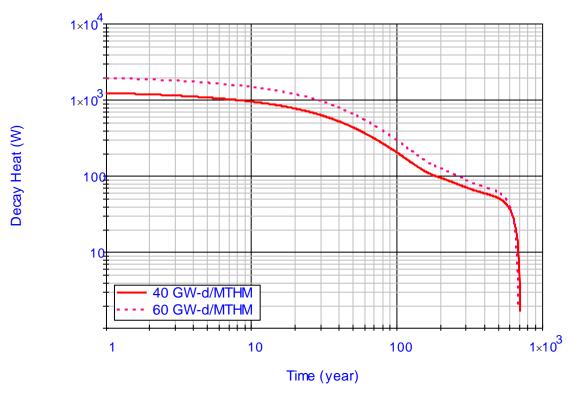


Figure 6. Decay Heat vs. Time Out of Reactor for Individual SNF Assemblies with Burnup of 40 and 60 GW-d/MTHM.

The results (Figures 7 to 10) show calculated maximum temperatures (peak waste package surface temperatures) for the different disposal concepts, as functions of initial power, waste package size, SNF burnup, and fuel age prior to disposal. The figures show strong correlation between maximum temperature and initial power, with slight shifts due to burnup and age.

For the salt calculation heat dissipation through the crushed salt backfill (comprising 1/4 of the package circumference) is ignored, i.e., the waste heat output is increased by 4/3, and the heat dissipates directly into intact salt. A similar approach was taken for salt in the original calculation (Hardin et al. 2001a, Section 5). The calculated maximum temperatures in salt (Figure 8) are greater than calculated using the FEM (Figure 4) because of this approximation. Thus, because of the more complex emplacement geometry of the generic salt repository, numerical analysis that includes backfill consolidation should be used (Clayton et al. 2012).

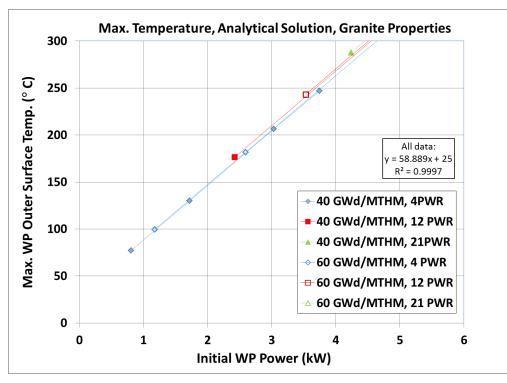


Figure 7. Maximum Temperature vs. Initial Power for Disposal in Crystalline Rock (with buffer) for Combinations of Waste Package Size, SNF Burnup, and Age

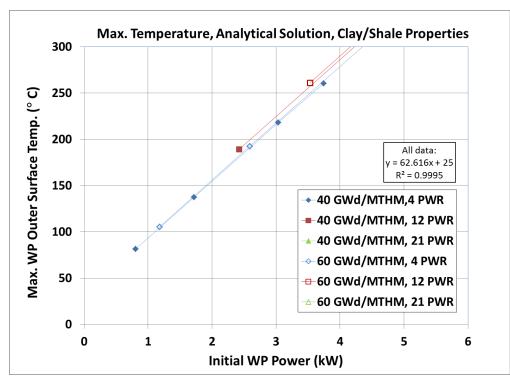


Figure 8. Maximum Temperature vs. Initial Power for Disposal in Clay/Shale (with buffer) for Combinations of Waste Package Size, SNF Burnup, and Age

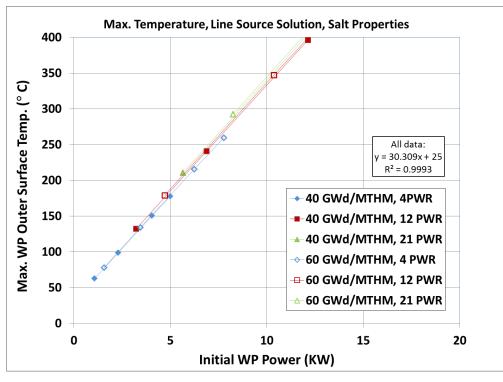


Figure 9. Maximum Temperature vs. Initial Power for Disposal in Salt (no buffer) for Combinations of Waste Package Size, SNF Burnup, and Age

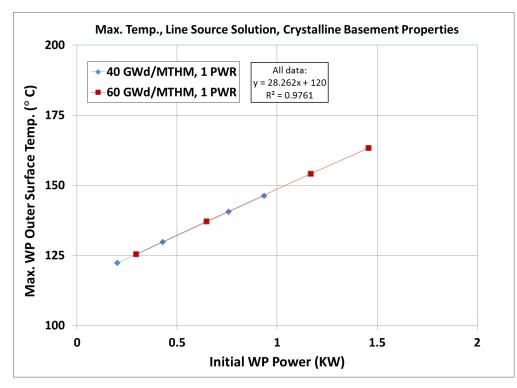


Figure 10. Maximum Temperature vs. Initial Power for Disposal in the Crystalline Basement (deep borehole concept; no buffer) for Combinations of Package Size, SNF Burnup, and Age

## 6. Summary and Conclusions

This report describes three contributions to analysis of uncertainty in maximum repository (peak waste package surface) temperatures:

- Analytical description of overall variance in temperature, as a function of contributions from key parameters of the analytical solution used in previous temperature analyses (Hardin et al. 2011a).
- Compilation of literature data on key parameter values, for various geologic host media and clay-based buffer materials, drawing on international work to develop geologic disposal solutions for heat-generating waste.
- Correlation between maximum repository temperature and waste package power at emplacement, without explicit adjustment for waste package size, SNF burnup, or fuel age.

The analytical treatment of temperature variance (Sections 3 and 4) uses partial derivatives of temperature with respect to each key parameter ( $K_{rock}$ ,  $K_{buf}$ ,  $\rho C_p$ , and  $r_2/r_1$ ), and separate variance estimates for the parameters, in a classical approach. The results include the following:

- Temperature at all times is relatively insensitive to heat capacitance (volumetric heat capacity) given the state of knowledge represented by the assigned parameter variance.
- Buffer thermal conductivity is an important parameter in early time (except for disposal concepts that have no buffer), but reported properties for dry and hydrated, clay-based buffer materials are relatively tightly grouped. Hence, the buffer radius ratio (expressed as  $r_2/r_1$ ) may be a more important parameter depending on how much it is allowed to vary in developing a disposal concept. This implies that disposal concepts that allow partial buffer hydration during the thermal period, are potentially increasing the greatest source of uncertainty in maximum temperature.
- Host rock thermal conductivity is the most important input parameter, especially at later time (e.g., greater than 10 years after emplacement) and where buffers are not used.

Compilation of literature data and selection of uncertainty ranges (Tables 1 through 5) are intended to provide mean and  $\pm 1\sigma$  property values for temperature uncertainty analysis based on the analytical solution (Equation 1).

Finally, the study of maximum (peak waste package surface) temperature (Section 5.2) corroborates the earlier finding from FEM simulations of the generic salt repository (Section 5.1). Thus the waste package heat output at emplacement can be used to predict maximum temperature, to a good approximation evident from the linearity in Figures 7 through 10, for all host media, waste package sizes, SNF burnup, and decay storage duration cases considered.

# References

Andra 2005a. *Dossier 2005 granite – architecture and management of a geological repository*. December, 2005. (www.Andra.fr/download/Andra-international-en/document/editions/ 285va.pdf).

Andra 2005b. *Dossier 2005 argile – architecture and management of a geological disposal system*. December 2005. (www.Andra.fr/international/download/Andra-international-en/document/editions/268va.pdf).

Bechthold, W., E. Smailos, S. Heusermann, W. Bollingerfehr, B. Bazargan Sabet, T. Rothfuchs, P. Kamlot, J. Grupa, S. Olivella and F.D. Hansen 2004. *Backfilling and Sealing of Underground Repositories for Radioactive Waste in Salt (BAMBUS II Project), Final Report.* EUR 20621, Nuclear Science and Technology, Luxembourg.

Brady, P.V., B.W. Arnold, G.A. Freeze, P.N. Swift, S.J. Bauer, J.L. Kanney, R.P. Rechard, and J.S. Stein 2009. *Deep borehole disposal of high-level radioactive waste*. SAND2009-4401. Albuquerque, NM: Sandia National Laboratories.

BSC (Bechtel-SAIC Co.) 2004. *Ventilation Model and Analysis Report*. U.S. Department of Energy, Office of Civilian Radioactive Waste Management. ANL-EBS-MD-000030 REV 04. October, 2004.

BSC (Bechtel-SAIC Co.) 2008a. *Basis of Design for the TAD Canister-Based Repository Design Concept*. U.S. Department of Energy, Office of Civilian Ra8dioactive Waste Management. 000-3DR-MGRO-00300-000-003. October, 2008.

BSC (Bechtel-SAIC Co.) 2008b. *Postclosure Analysis of the Range of Design Thermal Loadings*. U.S. Department of Energy, Office of Civilian Radioactive Waste Management. ANL-NBS-HS-000057 REV 00. January, 2008.

Carter, J.T., F. Hansen, R. Kehrman, and T. Hayes 2011. *A generic salt repository for disposal of waste from a spent nuclear fuel recycle facility*. SRNL-RP-2011-00149 Rev. 0. Aiken, SC: Savannah River National Laboratory.

Carter, J.T. and A.J. Luptak 2010. *Fuel Cycle Potential Waste Inventory for Disposition*. U.S. Department of Energy, Office of Used Fuel Disposition. FCR&D-USED-2010-000031 Rev 2. September, 2009.

Clayton, D.J., J.E. Bean, J.G. Arguello Jr., E.L. Hardin and F.D. Hansen 2012. "Thermalmechanical modeling of a generic high-level waste salt repository." In: SALTVII, 7th Conference on the Mechanical Behavior of Salt, Paris, France. April 16-19, 2012. (www.saltmech7.com).

Clayton, D.J. and C.W. Gable 2009. *3-D Thermal Analyses of High-Level Waste Emplaced in a Generic Salt Repository*. Sandia National Laboratories, Albuquerque, NM. SAND2009-0633P. February, 2009.

Fluor (Fluor Technology Inc.) 1986. Site Characterization Plan Conceptual Design Report for a High-Level Nuclear Waste Repository in Salt, Vertical Emplacement Mode. U.S. Department of Energy, Office of Civilian Radioactive Waste Management, Salt Repository Office. September, 1986.

Fluor (Fluor Technology Inc.) 1985. *Waste Package/Repository Impact Study: Final Report*. Battelle Memorial Institute, Office of Nuclear Waste Isolation, Columbus, OH. BMI/ONWI/C-312. September, 1985.

Gray, M.N. 1993. *OECD/NEA International Stripa Project, Overview Volume III. Engineered barriers.* Swedish Nuclear Fuel and Waste Management Co. (SKB).

Hahn, G.J. and S.S. Shapiro 1967. *Statistical Models in Engineering*. John Wiley & Sons, New York, N.Y.

Hardin, E., J. Blink, H. Greenberg, M. Sutton, M. Fratoni, J. Carter, M. Dupont and R. Howard 2011a. *Generic Repository Design Concepts and Thermal Analysis (FY11)*. FCRD-USED-2011-000143 Rev. 2. U.S. Department of Energy, Office of Used Fuel Disposition. December, 2011.

Hardin, E., D. Clayton, L. Zheng, J. Rutqvist, J. Houseworth, J. Davis, R. Tinnacher, L. Li, H.-H. Liu, M. Sutton, A. Tayne and T. Wolery 2011b. *Repository Science THMC Coupled Process Investigations (FY11)*. FCRD-USED-2011-000288 Rev. 0. U.S. Department of Energy, Office of Used Fuel Disposition. August, 2011.

Jia, Y., H.B. Bian, G. Duveau, K. Su and J.F. Shao 2009. "Numerical modelling of in situ behaviour of the Callovo–Oxfordian argillite subjected to the thermal loading," *Engineering Geology*. V.109, pp. 262–272.

Johnson, L.H., M. Niemeyer, G. Klubertanz, P. Siegel and P. Gribi 2002. *Calculations of the Temperature Evolution of a Repository for Spent Fuel, Vitrified High-Level Waste and Intermediate Level Waste in Opalinus Clay.* Nagra Tech Report 01-04. October, 2002.

Kreith, F. 1965. Principles of Heat Transfer (2nd edition). International Textbook Co.

Nagra (National Cooperative for the Disposal of Radioactive Waste) 1985. *Project Gewähr* 1985: Nuclear waste management in Switzerland: Feasibility studies and safety analyses. Nagra Project Report NGB 85-09.

ONDRAF/NIRAS (Belgian agency for radioactive waste and enriched fissile materials) 2001. *Technical overview of the SAFIR 2 report: Safety Assessment and Feasibility Interim Report 2*. NIROND 2001–05 E. December, 2001.

Pastina, B. and P. Hellä 2010. *Models and Data Report 2010*. Posiva OY, Olkiluoto, Finland. Posiva 2010-01, March, 2010.

Rohsenow, W.M., J.P. Hartnett and E.N. Ganic 1985. *Handbook of heat transfer fundamentals* (2nd edition). McGraw-Hill, New York, N.Y. 1440 pp.

Sillen, X. and J. Marivoet 2007. *Thermal impact of a HLW repository in clay: Deep disposal of vitrified high-level waste and spent fuel*. Belgian Nuclear Research Center, Mol, Belgium. SCK/CEN-ER-38, May, 2007.

SKB (Swedish Nuclear Fuel and Waste Management Co.) 2006. Long-term safety for KBS-3 repositories at Forsmark and Laxemar — A first evaluation. Technical Report TR-06-09.

Smyth, J.R., B. M. Crowe, P. M. Halleck and A. W. Reed 1979. A Preliminary Evaluation of the Radioactive Waste Isolation Potential of the Alluvium-Filled Valleys of the Great Basin. Los Alamos National Laboratory. LA-7962-MS.

Volckaert, G., F. Bernier and M. Dardaine 1996. *Demonstration of the in situ application of an industrial clay-based backfill material (BACCHUS 2)*. European Commission Report EUR 16860.

Weast R.C. (ed.) 1968. *CRC Handbook of Chemistry and Physics* (49th edition). CRC, The Chemical Rubber Company, Cleveland, Ohio.

Weetjens, E. and X. Sillen 2005. *Thermal analysis of the Supercontainer concept. 2D axisymmetric heat transport calculations*. SCK•CEN report R-4277.

Wollenberg, H.A., J.S.Y. Wang and G. Korbin 1983. An Appraisal of Nuclear Waste Isolation in the Vadose Zone in Arid and Semiarid Regions (with emphasis on the Nevada Test Site). U.S. Nuclear Regulatory Commission. NUREG CR-3158.

Wollenberg, H.A., J.S.Y. Wang and G. Korbin 1982. "Nuclear Waste Isolation in the Unsaturated Zone of Arid Regions." Presented at the Spring Meeting, American Geophysical Union, Philadelphia, May 31 - June 4, 1982. Lawrence Berkeley Laboratory, Berkeley, CA. LBL-14086.

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