

CRWMS/M&O

Calculation Cover Sheet

Complete only applicable items.

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CRC Depletion Calculations for Three Mile Island Unit 1

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10. Remarks
Attachments I through IV are contained on an attachment tape which was moved to Reference 7.7 as described in Section 8. Listings of the file content on the attachment tapes for Attachments I through IV are provided in their corresponding hard-copy attachment locations.
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- I: 1 page (file content listing)
- II: 29 pages (file content listing)
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- IV: 2 pages (file content listing)

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Table of Contents

<u>Item</u>	<u>Page</u>
1. Purpose.....	4
2. Method.....	4
3. Assumptions.....	4
4. Use of Computer Software	5
4.1. Software Approved for QA Work	5
4.1.1. SAS2H	5
4.2. Software Routines	5
4.2.1. Excel.....	6
4.2.2. CRAFT	6
4.2.3. CRC Data Tabulizer	6
4.2.4. RLAYOUT	6
5. Calculation	7
5.1. Three Mile Island Unit CRC Evaluation Description	7
5.2. Input Specifications for Depletion Calculations.....	7
5.2.1. Fuel Assembly Descriptions Required for Depletion Calculations.....	8
5.2.2. Axial Power Shaping Rod Assembly (APSRA) Descriptions	10
5.2.3. Rod Cluster Control Assembly (RCCA) Description Required for Depletion Calculations	10
5.2.4. System Pressure.....	11

5.2.5. Fuel Assembly Insertion, APSRA Insertion, and RCCA Insertion Histories for the Three Mile Island Unit 1 Depletion Calculations 11

5.2.6. Reactor Cycle History Specifications for Three Mile Island Unit 1 14

5.2.7. Boron Letdown Data for Three Mile Island Unit 1 Depletion Calculations... 15

5.2.8. Burnup, Fuel Temperature, and Moderator Specific Volume Data 16

5.2.9. RCCA Insertion History Data for Three Mile Island Unit 1 Depletion Calculations 38

5.3. Assembly Depletion Calculation Procedure 43

5.4. Path B Model Development for the Three Mile Island Unit 1 Depletion Calculations 43

5.5. Cycle Irradiation History Layouts for the Three Mile Island Unit 1 Depletion Calculations 59

5.6. The Commercial Reactor Assembly Follow Taskmaster (CRAFT) Software Routine and Usage 71

5.7. Input and Output Filename Descriptions for CRAFT and SAS2H 73

6. Results 74

7. References 75

8. Attachments 76

1. Purpose

The purpose of this calculation is to document the Three Mile Island Unit 1 pressurized water reactor (PWR) fuel depletion calculations performed as part of the commercial reactor critical (CRC) evaluation program. The CRC evaluations support the development and validation of the neutronics models used for criticality analyses involving commercial spent nuclear fuel in a geologic repository.

2. Method

The calculational method used to perform the Three Mile Island Unit 1 fuel depletion calculations consisted of using the SAS2H control sequence of the SCALE, Version 4.3, code system (Ref. 7.1) to deplete the necessary fuel assemblies. The various fuel assemblies were depleted through their unique operating histories such that their modified fuel compositions would be available at specific exposure times corresponding to the times (statepoints) at which detailed core reactivity calculations would be performed. The fuel assembly depletion calculations were based on detailed core follow information for each assembly.

3. Assumptions

- 3.1 The inherent approximation of uniformly distributed non-fuel lattice cells in the Path B models of the SAS2H calculations as described in Section 5.4 was considered acceptable within the fidelity of these calculations. The basis for this assumption was provided in Section S2.2.3.1 of Volume 1, Rev. 5 in Reference 7.1. This assumption was used throughout all of the depletion calculations documented in Section 5.
- 3.2 With the utilization of one cross section update per irradiation time step in the SAS2H calculations, the maximum duration of any time step in any reactor cycle irradiation layout should have not exceeded 80 days. The basis for this assumption was that the 80 day irradiation time step limit ensured that the changing isotopic concentrations of the fuel in the system would not alter the neutron spectrum radically enough to cause a time step of the depletion calculation to be performed without the availability of cross sections which have been properly weighted with an updated neutron spectrum and spatial flux. This assumption was used throughout all of the depletion calculations documented in Section 5.
- 3.3 Distributing the spacer grid material uniformly in the moderator composition of the SAS2H Path A and Path B models was acceptable. The basis for this assumption was that the limited reactivity worth of the spacer grid materials would have a negligible impact on the neutron spectrum when homogeneously distributed in the moderator. This assumption was used throughout all of the depletion calculations documented in Section 5.

4. Use of Computer Software

4.1. Software Approved for QA Work

4.1.1. SAS2H

The SAS2H control module of the SCALE, Version 4.3, modular code system (Ref. 7.1) was used to perform the fuel assembly depletion calculations required for the Three Mile Island Unit 1 CRC evaluations. The software specifications are as follow:

- Program Name: SAS2H of the SCALE Modular Code System
- Version/Revision Number: Version 4.3
- Computer Software Configuration Item (CSCI) Number: 30011 V4.3
- Computer Type: Hewlett Packard (HP) 9000 Series Workstations

The input and output files for the various SAS2H calculations were documented in the attachments to this calculation file as described in Section 5, such that an independent repetition of the software use could be performed. The SAS2H software used was: (a) appropriate for the application of commercial fuel assembly depletion, (b) used only within the range of validation as documented in References 7.1 and 7.2, (c) obtained from the Software Configuration Manager in accordance with appropriate procedures.

4.2. Software Routines

The description documentation for each of the software routines identified in this section, other than the acquired software routine Excel described in Section 4.2.1, contains the following information:

- Descriptions and equations of mathematical algorithms
- Description of software routine including execution environment
- Description of test cases
- Description of test results
- Range of input parameter values for which results were verified
- Identification of any limitations on software routine applications or validity
- Reference list of all documentation relevant to the qualification
- Directory listing of executable and data files
- Computer listing of source code
- Computer listing of test data input and output, identifying software routine name and version number.

4.2.1. Excel

- Title: Excel
- Version/Revision Number: Microsoft® Excel 97

The Excel spreadsheet program was used for simple numeric calculations as documented in Section 5 of this calculation file. The user-defined formulas, inputs, and results were documented in sufficient detail in Section 5 to allow an independent repetition of the various computations.

4.2.2. CRAFT

- Title: Commercial Reactor Assembly Follow Taskmaster (CRAFT)
- Version/Revision Number: Versions 4 and 5

The CRAFT software routine produced the input and directed the execution for the various SAS2H calculations required to deplete a commercial reactor fuel assembly to support a CRC evaluation. The input and output for the various CRAFT calculations were documented in Section 5, such that an independent repetition of the software routine use could be performed. The description of the CRAFT, Version 4 and 5, software routine was provided in Attachment I of Reference 7.6.

4.2.3. CRC Data Tabulizer

- Title: CRC_DATA_TABULIZER
- Version/Revision Number: Version 3

The CRC Data Tabulizer software routine produced tables containing the concentration results for a set of 29 isotopes and other relevant data at each CRC statepoint for a given fuel assembly. The CRC Data Tabulizer software routine is interactive, therefore, the input was not documented. However, the output contains all necessary information to verify that the input was provided correctly. The output from the CRC Data Tabulizer usage was provided in Attachment IV (this attachment has been moved to Reference 7.7). The information provided in this output and the information provided in the description of the CRC Data Tabulizer software routine, along with the CRAFT generated "*.cut" files, were sufficient such that an independent repetition of the software routine use could be performed. The description of the CRC Data Tabulizer, Version 3, software routine was provided in Attachment VI of Reference 7.6.

4.2.4. RLAYOUT

- Title: RLAYOUT
- Version/Revision Number: Version 1

The RLAYOUT software routine automated the development of irradiation time step layout inputs for depletion calculations involving rod insertion histories in which rod movements must be followed. The RLAYOUT code is mostly interactive, therefore, some of the input was not documented. The required boron letdown inputs and rod insertion history inputs for the required assemblies were presented in Sections 5.2.7 and 5.2.9, respectively. The output contained all necessary information to verify that the entire input was provided correctly. The output from the RLAYOUT usage was presented in Section 5.5. The information provided in this output, the boron letdown input, and the rod insertion history input along with the information provided in the description of the RLAYOUT software routine, are sufficient such that an independent repetition of the software routine use could be performed. The description of the RLAYOUT, Version 1, software routine was provided in Attachment III of Reference 7.3.

5. Calculation

5.1. Three Mile Island Unit 1 CRC Evaluation Description

The Three Mile Island Unit 1 CRC evaluations were performed at three statepoints (SP): Cycle 1, SP59 [0.0 Effective Full-Power Days (EFPD)], Cycle 5, SP60 and 61 [0.0 and 114.4 EFPD]. Each statepoint represented a specific time when the reactor was brought to the critical condition ($k_{\text{eff}} = 1$) and the corresponding reactor core conditions were measured. The CRC evaluations of each of these critical statepoints involved the use of SAS2H to deplete the various fuel assemblies and MCNP (Ref. 7.4) to model the reactor core such that the k_{eff} value at each of the critical statepoints could be predicted to demonstrate the ability of the dual code system. Hence, the objective of each CRC statepoint evaluation was to predict the reactor core k_{eff} as close to measurement as possible (the measurement is always $k_{\text{eff}} = 1$). The objective of the SAS2H depletion calculations documented in this calculation file was to provide the depleted fuel isotopic compositions to be used in the corresponding CRC reactivity calculations.

Fuel isotopic compositions were calculated with SAS2H for each depleted fuel assembly in each of the critical statepoint configurations to facilitate MCNP modeling. The Three Mile Island Unit 1 statepoint calculations required the depletion of fuel assemblies from four fuel batches. Fuel assembly design characteristics may vary between each fuel batch. Section 5.2 presents the input parameters required to perform the various fuel assembly depletion calculations. Sections 5.3 through 5.7 describe how the parameters listed in Section 5.2 were utilized to perform the SAS2H depletion calculations relevant to the CRC statepoint evaluations. The CRAFT description and user information provided in Attachment I of Reference 7.6 is essential for understanding the SAS2H modeling techniques employed in the calculations. The information provided in Attachment I of Reference 7.6, the input parameters provided in Section 5.2, and the CRAFT input decks contained in Attachment I (this attachment has been moved to Reference 7.7) work together to provide a complete description of how all of SAS2H depletion calculations were performed.

5.2. Input Specifications for Depletion Calculations

Title: CRC Depletion Calculations for Three Mile Island Unit 1
Document Identifier: B00000000-01717-0210-00007 REV 00

Page 8 of 76

The information documented in this section describes the design specifications and irradiation histories for the fuel assemblies required for the Three Mile Island Unit 1 CRC evaluations. All of the input specifications presented in this section were obtained from Reference 7.5. The Three Mile Island Unit 1 CRC evaluations included fuel assemblies from seven fuel batches identified as 1, 2, 3, 4, 5, 6 and 7. Depletion calculations for fuel assemblies from batches 4, 5, 6 and 7 were required to perform k_{eff} calculations at the various statepoints. Fuel assemblies from fuel batches 1, 2, and 3 were only used in the beginning-of-life (BOL) statepoint calculation. The BOL calculation included all fresh fuel (i.e., as fabricated, non-depleted).

5.2.1. Fuel Assembly Descriptions Required for Depletion Calculations

Table 5.2.1-1 contains a description of the fuel assemblies for fuel batches 4, 5, 6 and 7 of Three Mile Island Unit 1. All fuel assemblies within a given fuel batch have the same characteristics as presented in Table 5.2.1-1.

Title: CRC Depletion Calculations for Three Mile Island Unit 1
 Document Identifier: B00000000-01717-0210-00007 REV 00

Page 9 of 76

**Table 5.2.1-1. Fuel Assembly Descriptions for Batches 4 to 7 of Three Mile Island Unit 1
 (pp. 5, 22, Ref. 7.5)**

Parameter	Fuel Batch			
	4	5	6	7
Assembly Type	Mark-B4	Mark-B4	Mark-B4	Mark-B4
Weight Percent U-235	2.64	2.85	2.85	2.85
kg of U per Assembly	463.63	463.63	463.63	463.63
Fuel Height (cm)	360.172	360.172	360.172	360.172
Fuel Pellet OD ¹ (cm)	0.93980	0.93853	0.93853	0.93853
Fuel Rod Clad OD (cm)	1.0922	1.0922	1.0922	1.0922
Fuel Rod Clad ID ² (cm)	0.95758	0.95758	0.95758	0.95758
Spacer Grid Material	Inconel	Inconel	Inconel	Inconel
Volume Fraction of Spacer Grid in Moderator	0.005757609	0.005757609	0.005757609	0.005757609
Guide Tube Material	zircaloy	zircaloy	zircaloy	zircaloy
Guide Tube OD (cm)	1.3462	1.3462	1.3462	1.3462
Guide Tube ID (cm)	1.26492	1.26492	1.26492	1.26492
Instrument Tube Material	zircaloy	zircaloy	zircaloy	zircaloy
Instrument Tube OD (cm)	1.38193	1.38193	1.38193	1.38193
Instrument Tube ID (cm)	1.12014	1.12014	1.12014	1.12014
Array Size	15x15	15x15	15x15	15x15
Number of Fuel Rods	208	208	208	208
Number of Guide Tubes	16	16	16	16
Number of Instr. Tubes	1	1	1	1
Pin Pitch (cm)	1.44272	1.44272	1.44272	1.44272
Assembly Pitch (cm)	21.81098	21.81098	21.81098	21.81098

¹ OD = Outer Diameter

² ID = Inner Diameter

5.2.2. Axial Power Shaping Rod Assembly (APSRA) Description

Table 5.2.2-1 contains a description of the axial power shaping rod assemblies utilized in the various fuel assemblies from fuel batches 4, 5, 6 and 7 of Three Mile Island Unit 1. The rods of the APSRA are inserted, to a uniform height, into the guide tubes of the fuel assembly during irradiation to produce a local thermal flux depression.

Table 5.2.2-1. Black APSRA Description for Use in Batches 4 to 7 of Three Mile Island Unit 1 (p. 22, Ref. 7.5)

Parameter	Value
Axial Power Shaping Rod (APSR) Neutron Absorbing Material	Ag-In-Cd with a 79.8, 15.0, and 5.0 weight percent by mass composition, respectively
Ag-In-Cd Density (g/cc)	10.17
Absorber Pellet OD (cm)	0.99568
APSR Cladding Material	Stainless Steel 304 (SS304)
APSR Cladding OD (cm)	1.11760
APSR Cladding ID (cm)	1.01092
Number of APSRs in an APSRA	16

5.2.3. Rod Cluster Control Assembly (RCCA) Description Required for Depletion Calculations

The RCCA assemblies used in the Three Mile Island Unit 1 reactor were composed of 16 control rods (CRs) arranged in a "cluster" such that each guide tube in the fuel assembly could have a CR inserted from the top of the core to a uniform height in the assembly. Table 5.2.3-1 contains the description of the RCCAs utilized during the Three Mile Island Unit 1 reactor operation relevant to the CRC evaluations documented in this calculation file.

**Table 5.2.3-1. RCCA Descriptions for Use in Batch 4 to 7 of Three Mile Island Unit 1
 (p. 22, Ref. 7.5)**

Parameter	Value
Control Rod Neutron Absorbing Material	Ag-In-Cd with a 79.8, 15.0, and 5.0 weight percent by mass composition, respectively
Ag-In-Cd Density (g/cc)	10.17
Absorber Pellet OD (cm)	0.99568
Control Rod (CR) Cladding Material	Stainless Steel 304 (SS304)
CR Cladding OD (cm)	1.11760
CR Cladding ID (cm)	1.01092
Number of CR in a CRA	16

5.2.4. System Pressure

Three Mile Island Unit 1 is a B&W designed pressurized water reactor that operates at a constant pressure of 2200 psia (pounds per square inch absolute) (p. 5, Ref. 7.5).

5.2.5. Fuel Assembly Insertion, APSRA Insertion, and RCCA Insertion Histories for the Three Mile Island Unit 1 Depletion Calculations

The actual irradiation histories for the fuel assemblies from Three Mile Island Unit 1 were used to perform the SAS2H depletion calculations relevant to the CRC evaluations. Tables 5.2.5-1 and 5.2.5-2 identify the following information:

- the cycles in which the various fuel assemblies were inserted
- the locations of the various fuel assemblies in each cycle corresponding to a one-eighth core location as shown in Figure 5.2.5-1
- the fuel batch to which each fuel assembly corresponds
- the cycles in which the various fuel assemblies contained either a APSRA (Control Rod 8, (CR8)) or RCCA (Control Rod 7, (CR7))

Table 5.2.5-1. Assembly Position Histories for Fuel Assemblies from Batches 4 to 7 of Three Mile Island Unit 1 (p. 36, Ref. 7.5)

Assembly Number/Batch	Assembly Location In Cycle				
	1	2	3	4	5
Cycle 1 No assemblies from Cycle 1 are present in Cycle 5					
Cycle 2					
B15/4		K15	L12*	K12	K14
B21/4		L15	K12	N12*	H8
B21a/4		-	-	O13	N12*
B28/4		N14	M12	L11	L13
B29/4		O13	M11	H14*	H11
Cycle 3					
C8/5			H15	H12	H13
C15/5			K15	H10	H9
C15a/5			-	K9	M11
C20/5			L14	K11	K10
C21/5			L15	L12*	L11
C25/5			M14	M12	K12
C28/5			N14	K13	M12
C29/5			O13	L10*	O13
Cycle 4					
D8/6				H15	H10
D15/6				K15	K13
D20/6				L14	H12
D20a/6				-	H14*
D21/6				L15	L12*
D25/6				M14	K9
D25a/6				-	L10*
D27/6				N13	M13
D28/6				N14	K11
Cycle 5					
E8/7					H15
E15/7					K15
E20/7					L14
E21/7					L15
E25/7					M14
E27/7					N13
E28/7					N14

- * Contains RCCA
- Indicates the assembly was present in the cycle

	8	9	10	11	12	13	14	15
H	1	2	3	4	5	6	7	8
K		9	10	11	12	13	14	15
L			16	17	18	19	20	21
M				22	23	24	25	
N					26	27	28	
O						29		

Figure 5.2.5-1. One-Eighth Symmetric Core Layout for Three Mile Island Unit 1

**Table 5.2.5-2. Fuel Assemblies with RCCAs Inserted for Burnup Calculations
 (p. 37, Ref. 7.5)**

Assembly Number/Batch	Assembly Location In Cycle				
	1	2	3	4	5
Cycle 1 No assemblies from Cycle 1 are present in Cycle 5					
Cycle 2					
B15/4		- ³	CR8 ² /L12	-	-
B21/4		-	-	CR7 ¹ /N12	-
B21a/4		-	-	-	CR7/N12
B29/4		-	-	CR7/H14	-
Cycle 3					
C21/5			-	CR8/L12	-
C29/5			-	CR7/L10	-
Cycle 4					
D20a/6				-	CR7/H14
D21/6					CR8/L12
D25a/6				-	CR7/L10
Cycle 5 None of the assemblies inserted in Cycle 5 contained RCCA in Cycle 5					

¹CR7 contained RCCA
²CR8 contained APSR
³ - Indicates that the assembly was present in the cycle

5.2.6. Reactor Cycle History Specifications for Three Mile Island Unit 1

This section contains the Three Mile Island Unit 1 reactor cycle summary information relevant to the CRC evaluations documented in this calculation file. The calendar day duration between the various dates were determined using an Excel spreadsheet. Table 5.2.6-1 shows the cycle summary information. Table 5.2.6-2 shows the statepoint and datapoint summary information. The statepoints refer to times when the reactor was shutdown and restarted. MCNP reactivity calculations for the CRC evaluations were performed using the reactor startup conditions and appropriate depleted isotopics after each statepoint shutdown. The datapoints refer to times when the depletion calculations were halted to adjust various input parameters such as average fuel temperatures and average moderator specific volumes. The depletion calculations were continued after each datapoint halt without modeling any reactor downtime.

Table 5.2.6-1. Cycle Summary Information for Three Mile Island Unit 1 Depletion Calculations (pp. 23, 66, Ref. 7.5)

Cycle	Startup Date	Shutdown Date	Cycle Length (calendar days)	Cycle Length (EFPD)	Downtime at EOC ¹ (days)
1	06/05/74	02/21/76	626	467.4	93
2	05/24/76	03/18/77	298	256.2	56
3	05/13/77	03/17/78	308	287.1	42
4	04/28/78	02/17/79	295	274.0	2420
5	10/03/85	10/31/86	393	302.4	Not Required

¹ The letters "EOC" stand for end of cycle

Table 5.2.6-2. Statepoint and Datapoint Summary Information for Three Mile Island Unit 1 Depletion Calculations (pp. 40, 66, Ref. 7.5)

Cycle	EFPD	Statepoint or Datapoint Identifier	Downtime at Statepoint or Datapoint (hours)
1	0.0	SP59 ¹	0.0
2	0.0	DP1 ²	0.0
3	0.0	DP2	0.0
3	142.8	DP3	0.0
4	0.0	DP4	0.0
4	126.6	DP5	0.0
5	0.0	SP60	58080.0
5	114.4	SP61	773.7

¹ The letters "SP" refer to a CRC statepoint. The number immediately following the "SP" refers to the relative statepoint for the Three Mile Island Unit 1 CRC evaluations in the entire CRC evaluation project.

² The letters "DP" refer to a CRC datapoint. The number immediately following the "DP" refers to the relative datapoint for the Three Mile Island Unit 1 CRC evaluations.

5.2.7. Boron Letdown Data for Three Mile Island Unit 1 Depletion Calculations

The boron letdown data provided, on page 65 of Reference 7.5, for Cycles 2, 3, 4 and 5 of Three Mile Island Unit 1 is used to determine the soluble boron concentration in the moderator at the mid-point of each irradiation step in the various SAS2H depletion calculations performed to deplete the fuel assemblies. The boron concentrations at the irradiation step mid-point EFPD times are determined by

linear interpolation between the measured values listed in Table 5.2.7-1. The specific irradiation step mid-point boron concentrations obtained from this data are presented in Section 5.5.

Table 5.2.7-1. Boron Letdown Data for Cycles 2, 3, 4 and 5 of TMI Unit 1 (p. 65, Ref. 7.5)

Cycle 2		Cycle 3		Cycle 4		Cycle 5	
EFPD	ppmB ¹	EFPD	ppmB	EFPD	ppmB	EFPD	ppmB
25	812	25	688	25	765	25	749
50	740	50	617	50	694	50	677
75	667	75	545	75	624	75	604
100	595	100	473	100	554	100	532
125	522	125	402	125	483	125	459
150	450	150	330	150	413	150	386
175	377	175	258	175	343	175	314
200	305	200	177	200	273	200	241
225	232	225	115	225	202	225	168
250	160	250	43	250	132	250	96
256	142	287	10	274	64	275	23

¹ "ppmB" refers to parts per million by mass of natural boron in moderator (water).

5.2.8. Burnup, Fuel Temperature, and Moderator Specific Volume Data

Burnup, fuel temperature, and moderator specific volume data were required for each node of each assembly in each SAS2H depletion calculation. A set of nodal burnup data at the beginning and end of each SAS2H depletion calculation was required. A set of nodal fuel temperature and moderator specific volume data representative of full-power operation during each depletion calculation of interest (between statepoints and/or datapoints) was required. Tables 5.2.8-1 through 5.2.8-29 contain the burnup, fuel temperature, and moderator specific volume data required to perform all depletion calculations for each of the fuel assemblies present in the Three Mile Island Unit 1 CRC evaluations. The height of the top and bottom fuel assembly axial nodes in Tables 5.2.8-1 through 5.2.8-29 is 22.0660 cm, while the intermediate nodes are each 20.0025 cm high (p. 39, Ref. 7.5). These tables were obtained from pages 40 through 60 of Reference 7.5. The top of node 1 begins at the top of the active fuel region. The burnup data is presented in units of gigawatt-days per metric ton of uranium (GWd/MTU). The fuel temperature data is presented in units of degrees Fahrenheit. The moderator specific volume data is presented in units of cubic feet per pound. Each set of fuel temperature and moderator specific volume data listed in the tables was applicable to the depletion calculation performed between the statepoints and/or datapoints identified above the particular data.

Table 5.2.8-1. Burnup and Thermal Hydraulic Feedback Parameters for Assembly B15

Axial Node	Burnup DP1 to DP2			Burnup DP2 to DP3			Burnup DP3 to DP4		
	DP2	T-Fuel	Spec.Vol	DP3	T-Fuel	Spec.Vol	DP4	T-Fuel	Spec.Vol
1	2.757	846.7	0.0238	5.870	1091.7	0.0249	9.365	1079.5	0.0248
2	4.585	994.7	0.0238	9.542	1289.3	0.0248	14.670	1221.1	0.0247
3	5.563	1065.0	0.0237	11.485	1375.3	0.0246	17.246	1267.1	0.0245
4	6.003	1094.2	0.0236	12.368	1399.8	0.0244	18.303	1272.3	0.0243
5	6.184	1104.6	0.0235	12.731	1401.0	0.0242	18.697	1266.7	0.0241
6	6.253	1107.6	0.0234	12.849	1393.3	0.0240	18.819	1260.8	0.0239
7	6.276	1107.9	0.0233	12.814	1379.6	0.0238	18.775	1256.1	0.0238
8	6.276	1107.3	0.0232	12.609	1345.5	0.0236	18.449	1244.9	0.0236
9	6.264	1106.1	0.0231	11.814	1225.0	0.0234	16.972	1189.1	0.0234
10	6.249	1104.8	0.0230	10.002	1060.4	0.0233	13.893	1040.0	0.0233
11	6.247	1104.3	0.0229	9.694	1033.9	0.0232	13.262	1014.8	0.0232
12	6.266	1105.0	0.0228	9.740	1037.4	0.0231	13.302	1014.4	0.0231
13	6.300	1106.4	0.0227	10.007	1069.2	0.0230	13.771	1032.8	0.0230
14	6.323	1106.8	0.0226	11.147	1233.9	0.0229	15.957	1136.6	0.0229
15	6.279	1102.1	0.0225	12.458	1363.5	0.0228	18.348	1246.9	0.0228
16	6.014	1080.9	0.0225	12.293	1370.6	0.0226	18.288	1264.0	0.0226
17	5.150	1014.8	0.0224	10.694	1300.2	0.0225	16.170	1228.5	0.0225
18	3.125	849.5	0.0223	6.669	1085.3	0.0224	10.419	1072.4	0.0224

Axial Node	Burnup DP4 to DP5			Burnup DP5 to SP60			Burnup SP60 to SP61		
	DP5	T-Fuel	Spec.Vol	SP60	T-Fuel	Spec.Vol	SP61	T-Fuel	Spec.Vol
1	11.740	970.0	0.0247	14.84	983.3	0.0246	16.242	810.3	0.0229
2	18.319	1090.7	0.0246	22.77	1068.7	0.0245	24.968	879.8	0.0228
3	21.518	1131.1	0.0244	26.493	1089.2	0.0244	29.145	900.6	0.0228
4	22.846	1143.8	0.0243	27.957	1086.9	0.0242	30.836	905.3	0.0227
5	23.343	1145.5	0.0241	28.464	1079.5	0.0241	31.439	904.5	0.0226
6	23.474	1141.9	0.0239	28.584	1073.0	0.0239	31.589	899.8	0.0225
7	23.368	1135.2	0.0238	28.476	1069.2	0.0238	31.484	894.7	0.0224
8	22.910	1123.3	0.0236	28.015	1067.3	0.0236	31.039	895.3	0.0224
9	21.377	1122.0	0.0235	26.485	1073.4	0.0235	29.614	916.2	0.0223
10	18.528	1162.7	0.0233	23.714	1098.9	0.0233	27.073	962.5	0.0222
11	18.032	1170.5	0.0232	23.188	1100.9	0.0232	26.627	980.7	0.0221
12	18.221	1172.5	0.0230	23.381	1100.8	0.0231	26.828	986.4	0.0220
13	18.821	1172.9	0.0229	24.012	1100.7	0.0229	27.399	979.2	0.0219
14	20.861	1142.2	0.0228	26.018	1085.4	0.0228	29.187	942.2	0.0218
15	22.834	1085.8	0.0226	27.874	1061.1	0.0227	30.765	898.0	0.0218
16	22.357	1046.3	0.0225	27.254	1050.6	0.0225	29.908	871.8	0.0217
17	19.610	995.9	0.0224	24.028	1024.2	0.0224	26.304	843.0	0.0216
18	12.643	883.1	0.0223	15.712	934.8	0.0223	17.211	776.2	0.0216

Data point or

Statepoint	EFPD / Cycle
DP1	0.0 / Cy2
DP2	0.0 / Cy3
DP3	142.8 / Cy3
DP4	0.0 / Cy4
DP5	126.6 / Cy4
SP60	0.0 / Cy5
SP61	114.4 / Cy5

Burnup	- GWd/MTU
T-Fuel	- °F
Spec. Vol.	- ft ³ / lbm

Table 5.2.8-2. Burnup and Thermal Hydraulic Parameters for Assembly B21

Axial Node	Burnup DP1 to DP2			Burnup DP2 to DP3			Burnup DP3 to DP4		
	DP2	T-Fuel	Spec.Vol	DP3	T-Fuel	Spec.Vol	DP4	T-Fuel	Spec.Vol
1	2.237	797.4	0.0236	5.158	1084.4	0.0254	8.567	1073.8	0.0252
2	3.781	923.6	0.0235	8.540	1297.0	0.0253	13.644	1224.0	0.0251
3	4.643	988.8	0.0235	10.511	1399.7	0.0251	16.320	1283.3	0.0249
4	5.054	1017.0	0.0234	11.472	1429.5	0.0248	17.472	1293.3	0.0247
5	5.235	1027.7	0.0233	11.883	1431.8	0.0246	17.912	1288.7	0.0245
6	5.310	1031.3	0.0232	12.037	1425.6	0.0244	18.068	1283.1	0.0243
7	5.336	1031.9	0.0231	12.053	1414.6	0.0242	18.081	1278.8	0.0241
8	5.334	1031.3	0.0230	11.943	1396.0	0.0240	17.928	1273.0	0.0239
9	5.315	1029.7	0.0230	11.653	1361.1	0.0238	17.494	1258.2	0.0237
10	5.289	1027.6	0.0229	11.268	1325.5	0.0236	16.914	1235.9	0.0235
11	5.272	1026.1	0.0228	11.106	1314.8	0.0234	16.671	1226.3	0.0234
12	5.279	1026.0	0.0227	11.151	1323.4	0.0233	16.739	1227.0	0.0232
13	5.305	1027.0	0.0227	11.347	1348.5	0.0231	17.048	1236.4	0.0231
14	5.327	1027.7	0.0226	11.676	1387.6	0.0229	17.602	1257.6	0.0229
15	5.285	1023.5	0.0225	11.874	1409.4	0.0228	17.985	1278.5	0.0228
16	5.033	1003.8	0.0224	11.469	1397.2	0.0226	17.517	1280.0	0.0226
17	4.253	942.6	0.0224	9.880	1320.9	0.0225	15.379	1240.2	0.0225
18	2.533	798.9	0.0223	6.117	1095.6	0.0224	9.876	1080.6	0.0224

Axial Node	Burnup DP4 to DP5			Burnup DP5 to SP60			Burnup SP60 to SP61		
	DP5	T-Fuel	Spec.Vol	SP60	T-Fuel	Spec.Vol	SP61	T-Fuel	Spec.Vol
1	9.976	841.0	0.0246	12.029	889.9	0.0246	14.202	922.7	0.0230
2	16.638	1057.0	0.0245	20.726	1051.2	0.0245	23.749	976.5	0.0229
3	20.312	1124.1	0.0244	25.144	1083.8	0.0243	28.438	973.4	0.0228
4	21.909	1144.0	0.0242	26.931	1083.8	0.0242	30.288	959.2	0.0227
5	22.525	1150.5	0.0240	27.594	1078.5	0.0240	30.942	944.5	0.0226
6	22.729	1150.2	0.0239	27.809	1073.4	0.0239	31.123	931.9	0.0225
7	22.710	1144.7	0.0237	27.796	1069.9	0.0237	31.076	923.0	0.0224
8	22.469	1134.9	0.0236	27.552	1067.5	0.0236	30.813	919.8	0.0224
9	21.945	1124.1	0.0234	26.999	1066.0	0.0234	30.265	923.8	0.0223
10	21.327	1116.8	0.0233	26.329	1064.8	0.0233	29.612	932.0	0.0222
11	21.095	1110.7	0.0231	26.046	1062.5	0.0232	29.324	936.9	0.0221
12	21.208	1106.2	0.0230	26.136	1060.5	0.0230	29.378	936.4	0.0220
13	21.561	1103.0	0.0229	26.502	1060.1	0.0229	29.682	930.2	0.0219
14	22.084	1095.2	0.0227	27.061	1060.1	0.0228	30.156	919.0	0.0219
15	22.300	1077.1	0.0226	27.285	1059.0	0.0227	30.288	907.7	0.0218
16	21.494	1047.2	0.0225	26.361	1053.0	0.0225	29.247	899.2	0.0217
17	18.730	995.3	0.0224	23.104	1025.7	0.0224	25.714	884.1	0.0216
18	12.000	874.7	0.0223	14.990	929.7	0.0223	16.813	821.7	0.0216

Data point or

Statepoint	EFPD / Cycle
DP1	0.0 / Cy2
DP2	0.0 / Cy3
DP3	142.8 / Cy3
DP4	0.0 / Cy4
DP5	126.6 / Cy4
SP60	0.0 / Cy5
SP61	114.4 / Cy5

Burnup - GWd/MTU
 T-Fuel - °F
 Spec. Vol. - ft³ / lbm

Table 5.2.8-3. Burnup and Thermal Hydraulic Parameters for Assembly B21a

Axial Node	Burnup DP1 to DP2			Burnup DP2 to DP3			Burnup DP3 to DP4		
	DP2	T-Fuel	Spec.Vol	DP3	T-Fuel	Spec.Vol	DP4	T-Fuel	Spec.Vol
1	2.237	797.4	0.0236	5.158	1084.4	0.0254	8.567	1073.8	0.0252
2	3.781	923.6	0.0235	8.540	1297.0	0.0253	13.644	1224.0	0.0251
3	4.643	988.8	0.0235	10.511	1399.7	0.0251	16.320	1283.3	0.0249
4	5.054	1017.0	0.0234	11.472	1429.5	0.0248	17.472	1293.3	0.0247
5	5.235	1027.7	0.0233	11.883	1431.8	0.0246	17.912	1288.7	0.0245
6	5.310	1031.3	0.0232	12.037	1425.6	0.0244	18.068	1283.1	0.0243
7	5.336	1031.9	0.0231	12.053	1414.6	0.0242	18.081	1278.8	0.0241
8	5.334	1031.3	0.0230	11.943	1396.0	0.0240	17.928	1273.0	0.0239
9	5.315	1029.7	0.0230	11.653	1361.1	0.0238	17.494	1258.2	0.0237
10	5.289	1027.6	0.0229	11.268	1325.5	0.0236	16.914	1235.9	0.0235
11	5.272	1026.1	0.0228	11.106	1314.8	0.0234	16.671	1226.3	0.0234
12	5.279	1026.0	0.0227	11.151	1323.4	0.0233	16.739	1227.0	0.0232
13	5.305	1027.0	0.0227	11.347	1348.5	0.0231	17.048	1236.4	0.0231
14	5.327	1027.7	0.0226	11.676	1387.6	0.0229	17.602	1257.6	0.0229
15	5.285	1023.5	0.0225	11.874	1409.4	0.0228	17.985	1278.5	0.0228
16	5.033	1003.8	0.0224	11.469	1397.2	0.0226	17.517	1280.0	0.0226
17	4.253	942.6	0.0224	9.880	1320.9	0.0225	15.379	1240.2	0.0225
18	2.533	798.9	0.0223	6.117	1095.6	0.0224	9.876	1080.6	0.0224

Axial Node	Burnup DP4 to DP5			Burnup DP5 to SP60			Burnup SP60 to SP61		
	DP5	T-Fuel	Spec.Vol	SP60	T-Fuel	Spec.Vol	SP61	T-Fuel	Spec.Vol
1	9.550	756.3	0.0234	10.976	782.0	0.0235	12.189	832.6	0.0231
2	15.297	832.5	0.0234	17.527	850.5	0.0234	19.792	979.3	0.0230
3	18.381	869.0	0.0233	20.998	869.9	0.0234	24.170	1009.8	0.0229
4	19.753	884.1	0.0232	22.507	872.4	0.0233	26.019	1012.2	0.0228
5	20.298	889.7	0.0232	23.091	870.4	0.0232	26.710	1006.4	0.0227
6	20.491	890.4	0.0231	23.295	867.7	0.0231	26.925	995.4	0.0226
7	20.504	888.5	0.0230	23.313	865.9	0.0231	26.892	980.7	0.0225
8	20.333	885.5	0.0229	23.150	865.4	0.0230	26.650	968.6	0.0225
9	19.887	883.6	0.0229	22.713	867.0	0.0229	26.159	968.2	0.0224
10	19.310	883.1	0.0228	22.144	869.8	0.0228	25.592	980.1	0.0223
11	19.075	881.0	0.0227	21.905	870.5	0.0228	25.390	995.7	0.0222
12	19.147	877.5	0.0227	21.969	869.5	0.0227	25.511	1011.0	0.0221
13	19.444	872.1	0.0226	22.258	867.4	0.0226	25.844	1019.7	0.0220
14	19.942	862.5	0.0225	22.741	863.7	0.0226	26.312	1014.7	0.0219
15	20.210	848.2	0.0225	22.970	858.9	0.0225	26.446	999.1	0.0218
16	19.542	828.1	0.0224	22.190	851.5	0.0224	25.466	977.3	0.0217
17	17.047	793.8	0.0224	19.361	831.0	0.0224	22.209	939.4	0.0216
18	10.897	724.1	0.0223	12.413	764.2	0.0223	14.290	844.2	0.0216

Data point or

Statepoint	FFPD / Cycle
DP1	0.0 / Cy2
DP2	0.0 / Cy3
DP3	142.8 / Cy3
DP4	0.0 / Cy4
DP5	126.6 / Cy4
SP60	0.0 / Cy5
SP61	114.4 / Cy5

Burnup	- GWd/MTU
T-Fuel	- °F
Spec. Vol.	- ft ³ / lbm

Table 5.2.8-4. Burnup and Thermal Hydraulic Parameters for Assembly B28

Axial Node	Burnup DP1 to DP2			Burnup DP2 to DP3			Burnup DP3 to DP4		
	DP2	T-Fuel	Spec.Vol	DP3	T-Fuel	Spec.Vol	DP4	T-Fuel	Spec.Vol
1	2.423	813.7	0.0237	5.230	1048.0	0.0249	8.376	1050.4	0.0249
2	4.091	946.4	0.0237	8.553	1231.6	0.0248	13.187	1181.3	0.0247
3	5.082	1017.6	0.0236	10.377	1306.8	0.0246	15.596	1224.8	0.0246
4	5.638	1054.6	0.0235	11.285	1326.9	0.0244	16.686	1228.1	0.0244
5	5.935	1072.4	0.0234	11.707	1324.9	0.0242	17.179	1221.1	0.0242
6	6.073	1079.8	0.0233	11.866	1315.1	0.0240	17.370	1214.4	0.0240
7	6.126	1082.0	0.0232	11.864	1300.0	0.0239	17.373	1209.3	0.0239
8	6.129	1081.6	0.0231	11.705	1272.9	0.0237	17.159	1201.5	0.0237
9	6.099	1079.4	0.0231	11.326	1228.6	0.0235	16.606	1182.7	0.0236
10	6.051	1076.1	0.0230	10.828	1183.7	0.0234	15.870	1155.0	0.0234
11	6.013	1073.2	0.0229	10.575	1166.0	0.0233	15.505	1142.3	0.0233
12	6.007	1072.2	0.0228	10.583	1172.7	0.0232	15.533	1143.6	0.0232
13	6.033	1073.1	0.0227	10.817	1203.1	0.0230	15.913	1158.0	0.0230
14	6.061	1074.4	0.0226	11.278	1260.9	0.0229	16.690	1192.0	0.0229
15	6.013	1070.8	0.0225	11.727	1318.6	0.0227	17.499	1235.3	0.0228
16	5.718	1049.8	0.0225	11.634	1343.5	0.0226	17.528	1257.5	0.0226
17	4.836	984.4	0.0224	10.182	1286.9	0.0225	15.610	1227.7	0.0225
18	2.901	828.8	0.0223	6.337	1076.4	0.0223	10.049	1071.5	0.0224

Axial Node	Burnup DP4 to DP5			Burnup DP5 to SP60			Burnup SP60 to SP61		
	DP5	T-Fuel	Spec.Vol	SP60	T-Fuel	Spec.Vol	SP61	T-Fuel	Spec.Vol
1	10.742	976.7	0.0247	13.826	989.8	0.0246	15.701	876.0	0.0230
2	16.951	1122.0	0.0246	21.527	1090.8	0.0245	24.348	953.3	0.0230
3	20.111	1175.6	0.0244	25.290	1115.9	0.0244	28.550	973.5	0.0229
4	21.531	1188.2	0.0243	26.852	1112.8	0.0242	30.304	974.6	0.0228
5	22.139	1187.7	0.0241	27.461	1103.5	0.0240	30.984	967.9	0.0227
6	22.335	1182.2	0.0239	27.638	1095.7	0.0239	31.152	956.2	0.0226
7	22.255	1173.3	0.0237	27.548	1090.8	0.0237	30.980	936.4	0.0225
8	21.859	1156.2	0.0236	27.122	1086.0	0.0236	30.411	913.7	0.0224
9	21.089	1131.2	0.0234	26.225	1076.1	0.0234	29.405	914.1	0.0223
10	20.262	1116.5	0.0233	25.193	1064.0	0.0233	28.379	931.9	0.0222
11	19.916	1110.2	0.0232	24.732	1057.5	0.0232	27.984	952.5	0.0222
12	20.031	1106.8	0.0230	24.809	1054.6	0.0230	28.165	976.1	0.0221
13	20.541	1108.8	0.0229	25.351	1055.6	0.0229	28.812	993.3	0.0220
14	21.377	1112.9	0.0228	26.315	1063.2	0.0228	29.792	984.8	0.0219
15	22.014	1094.0	0.0226	27.043	1065.9	0.0227	30.409	961.4	0.0218
16	21.671	1058.4	0.0225	26.603	1057.9	0.0225	29.751	935.2	0.0217
17	19.136	1008.4	0.0224	23.605	1031.7	0.0224	26.338	900.8	0.0216
18	12.338	892.7	0.0223	15.454	941.3	0.0223	17.276	822.6	0.0216

Data point or

Statepoint	EFPD / Cycle
DP1	0.0 / Cy2
DP2	0.0 / Cy3
DP3	142.8 / Cy3
DP4	0.0 / Cy4
DP5	126.6 / Cy4
SP60	0.0 / Cy5
SP61	114.4 / Cy5

Burnup	- GWd/MTU
T-Fuel	- °F
Spec. Vol.	- ft ³ / lbm

Table 5.2.8-5. Burnup and Thermal Hydraulic Parameters for Assembly B29

Axial Node	Burnup DP1 to DP2			Burnup DP2 to DP3			Burnup DP3 to DP4		
	DP2	T-Fuel	Spec.Vol	DP3	T-Fuel	Spec.Vol	DP4	T-Fuel	Spec.Vol
1	2.490	820.8	0.0239	5.521	1088.1	0.0252	8.927	1075.1	0.0251
2	4.195	955.5	0.0238	9.033	1285.6	0.0250	14.048	1215.4	0.0249
3	5.278	1033.1	0.0237	11.056	1367.8	0.0248	16.687	1260.6	0.0247
4	5.985	1079.8	0.0236	12.161	1386.3	0.0246	17.941	1261.0	0.0245
5	6.387	1104.5	0.0235	12.696	1379.9	0.0244	18.493	1251.1	0.0243
6	6.564	1114.1	0.0234	12.890	1367.7	0.0242	18.685	1242.9	0.0242
7	6.624	1116.6	0.0233	12.892	1350.2	0.0240	18.674	1237.2	0.0240
8	6.628	1116.1	0.0232	12.731	1322.5	0.0238	18.453	1229.7	0.0238
9	6.596	1113.8	0.0231	12.375	1282.2	0.0237	17.946	1214.2	0.0236
10	6.548	1110.6	0.0230	11.944	1244.3	0.0235	17.330	1193.5	0.0235
11	6.510	1107.7	0.0229	11.714	1229.3	0.0234	17.008	1183.0	0.0233
12	6.505	1106.7	0.0228	11.732	1237.4	0.0232	17.050	1184.1	0.0232
13	6.532	1107.6	0.0227	11.964	1267.2	0.0231	17.419	1196.3	0.0231
14	6.559	1108.7	0.0226	12.362	1315.6	0.0229	18.071	1221.2	0.0229
15	6.509	1105.1	0.0226	12.674	1356.8	0.0228	18.629	1249.1	0.0228
16	6.207	1084.5	0.0225	12.417	1363.4	0.0226	18.405	1260.9	0.0226
17	5.286	1019.1	0.0224	10.837	1299.7	0.0225	16.340	1229.5	0.0225
18	3.209	854.2	0.0223	6.783	1087.7	0.0224	10.565	1074.9	0.0224

Axial Node	Burnup DP4 to DP5			Burnup DP5 to SP60			Burnup SP60 to SP61		
	DP5	T-Fuel	Spec.Vol	SP60	T-Fuel	Spec.Vol	SP61	T-Fuel	Spec.Vol
1	10.033	788.9	0.0241	11.754	844.1	0.0242	14.040	947.2	0.0232
2	16.385	961.4	0.0241	19.833	987.9	0.0242	23.105	1018.0	0.0231
3	19.790	1014.5	0.0240	23.880	1017.2	0.0240	27.519	1025.8	0.0230
4	21.372	1027.9	0.0238	25.623	1016.7	0.0239	29.375	1014.7	0.0229
5	22.050	1031.3	0.0237	26.338	1011.4	0.0238	30.097	999.2	0.0228
6	22.288	1031.0	0.0236	26.584	1006.8	0.0237	30.304	983.3	0.0227
7	22.293	1029.7	0.0235	26.604	1004.7	0.0235	30.260	968.3	0.0226
8	22.089	1029.9	0.0234	26.432	1006.0	0.0234	30.026	958.6	0.0225
9	21.639	1035.1	0.0232	26.040	1011.7	0.0233	29.603	959.6	0.0224
10	21.130	1045.0	0.0231	25.610	1020.6	0.0232	29.180	969.3	0.0223
11	20.943	1054.9	0.0230	25.505	1028.4	0.0231	29.091	979.8	0.0222
12	21.094	1059.9	0.0229	25.720	1032.9	0.0230	29.314	987.2	0.0221
13	21.462	1053.2	0.0228	26.097	1031.2	0.0228	29.678	988.2	0.0220
14	21.966	1032.0	0.0227	26.541	1022.6	0.0227	30.075	980.7	0.0219
15	22.240	999.2	0.0226	26.684	1009.0	0.0226	30.135	968.7	0.0218
16	21.607	958.4	0.0225	25.801	990.3	0.0225	29.114	956.2	0.0217
17	18.958	906.2	0.0224	22.625	957.9	0.0224	25.595	932.4	0.0217
18	12.214	808.7	0.0223	14.691	871.7	0.0223	16.729	854.6	0.0216

Data point or

Statepoint	EFPD / Cycle
DP1	0.0 / Cy2
DP2	0.0 / Cy3
DP3	142.8 / Cy3
DP4	0.0 / Cy4
DP5	126.6 / Cy4
SP60	0.0 / Cy5
SP61	114.4 / Cy5

Burnup - GWd/MTU
 T-Fuel - °F
 Spec. Vol. - ft³ / lbm

Table 5.2.8-6. Burnup and Thermal Hydraulic Parameters for Assembly C8

Axial Node	Burnup DP2 to DP3			Burnup DP3 to DP4			Burnup DP4 to DP5		
	DP3	T-Fuel	Spec.Vol	DP4	T-Fuel	Spec.Vol	DP5	T-Fuel	Spec.Vol
1	1.448	841.6	0.0240	3.243	888.1	0.0241	5.837	1069.4	0.0251
2	2.482	998.5	0.0239	5.364	1034.0	0.0240	9.455	1242.4	0.0250
3	3.110	1083.9	0.0239	6.549	1094.6	0.0239	11.389	1313.1	0.0248
4	3.452	1125.2	0.0238	7.174	1115.2	0.0238	12.335	1335.4	0.0246
5	3.633	1143.3	0.0236	7.537	1120.4	0.0237	12.814	1336.8	0.0244
6	3.722	1149.3	0.0235	7.741	1121.2	0.0236	13.035	1331.2	0.0242
7	3.761	1149.2	0.0234	7.853	1121.4	0.0235	13.108	1322.4	0.0240
8	3.772	1146.8	0.0233	7.920	1122.1	0.0234	13.118	1311.3	0.0238
9	3.771	1144.7	0.0232	7.972	1123.4	0.0233	13.168	1302.8	0.0236
10	3.776	1145.1	0.0231	8.028	1125.4	0.0232	13.327	1302.2	0.0235
11	3.803	1149.7	0.0230	8.106	1128.3	0.0231	13.565	1306.7	0.0233
12	3.855	1158.8	0.0229	8.208	1132.5	0.0230	13.829	1312.5	0.0231
13	3.930	1171.6	0.0228	8.332	1139.4	0.0229	14.013	1311.7	0.0230
14	4.033	1189.3	0.0227	8.504	1153.5	0.0227	14.047	1295.4	0.0228
15	4.173	1213.2	0.0226	8.746	1177.0	0.0226	13.942	1257.8	0.0227
16	4.200	1219.3	0.0225	8.741	1185.2	0.0225	13.418	1201.3	0.0225
17	3.631	1138.4	0.0224	7.626	1134.0	0.0224	11.520	1120.8	0.0224
18	2.161	920.5	0.0223	4.683	958.9	0.0223	7.126	961.2	0.0223

Axial Node	Burnup DP5 to SP60			Burnup SP60 to SP61		
	SP60	T-Fuel	Spec.Vol	SP60	T-Fuel	Spec.Vol
1	9.231	1068.6	0.0250	11.220	930.2	0.0233
2	14.442	1201.0	0.0249	17.528	1050.0	0.0232
3	17.009	1239.6	0.0247	20.648	1082.1	0.0231
4	18.120	1239.3	0.0245	21.990	1083.8	0.0230
5	18.610	1229.6	0.0243	22.552	1074.8	0.0228
6	18.817	1220.7	0.0241	22.754	1061.7	0.0227
7	18.887	1215.0	0.0240	22.779	1047.8	0.0226
8	18.904	1211.3	0.0238	22.744	1037.7	0.0225
9	18.966	1209.1	0.0236	22.781	1036.9	0.0224
10	19.141	1208.1	0.0235	22.967	1044.1	0.0223
11	19.408	1208.7	0.0233	23.253	1053.6	0.0222
12	19.716	1210.8	0.0232	23.573	1061.2	0.0221
13	19.928	1212.5	0.0230	23.772	1063.6	0.0220
14	19.948	1211.6	0.0229	23.736	1057.5	0.0219
15	19.760	1205.0	0.0227	23.435	1043.2	0.0218
16	19.014	1188.9	0.0226	22.492	1022.6	0.0217
17	16.515	1147.6	0.0224	19.576	986.3	0.0217
18	10.509	1008.9	0.0223	12.542	877.0	0.0216

Datapoint
or

Statepoint	EFPD / Cycle
DP2	0.0 / Cy3
DP3	142.8 / Cy3
DP4	0.0 / Cy4
DP5	126.6 / Cy4
SP60	0.0 / Cy5
SP61	114.4 / Cy5

Burnup	- GWd/MTU
T-Fuel	- °F
Spec. Vol.	- ft ³ / lbm

Table 5.2.8-7. Burnup and Thermal Hydraulic Parameters for Assembly C15

Axial Node	Burnup DP2 to DP3			Burnup DP3 to DP4			Burnup DP4 to DP5		
	DP3	T-Fuel	Spec.Vol	DP4	T-Fuel	Spec.Vol	DP5	T-Fuel	Spec.Vol
1	1.575	863.7	0.0242	3.506	909.0	0.0242	6.223	1080.1	0.0251
2	2.714	1033.8	0.0241	5.815	1060.6	0.0242	10.072	1253.6	0.0250
3	3.412	1126.1	0.0240	7.108	1124.5	0.0240	12.134	1325.5	0.0248
4	3.795	1172.0	0.0239	7.772	1145.4	0.0239	13.132	1346.5	0.0246
5	3.997	1191.2	0.0238	8.130	1150.8	0.0238	13.616	1347.1	0.0244
6	4.098	1197.5	0.0236	8.330	1151.8	0.0237	13.841	1340.6	0.0242
7	4.143	1197.6	0.0235	8.443	1152.3	0.0236	13.933	1331.7	0.0240
8	4.155	1195.0	0.0234	8.510	1153.2	0.0234	13.962	1321.2	0.0238
9	4.151	1192.4	0.0233	8.557	1154.7	0.0233	13.981	1310.6	0.0236
10	4.153	1192.5	0.0232	8.608	1156.7	0.0232	14.028	1301.2	0.0235
11	4.178	1197.1	0.0231	8.682	1159.4	0.0231	14.117	1293.2	0.0233
12	4.232	1206.7	0.0230	8.785	1163.4	0.0230	14.229	1285.5	0.0231
13	4.313	1220.3	0.0228	8.912	1169.6	0.0229	14.318	1275.5	0.0230
14	4.406	1236.3	0.0227	9.058	1180.0	0.0228	14.342	1259.8	0.0228
15	4.471	1248.7	0.0226	9.158	1192.0	0.0226	14.213	1234.8	0.0227
16	4.342	1233.3	0.0225	8.909	1188.2	0.0225	13.587	1196.4	0.0225
17	3.673	1141.5	0.0224	7.662	1132.9	0.0224	11.649	1128.8	0.0224
18	2.164	920.3	0.0223	4.672	956.7	0.0223	7.211	972.8	0.0223

Axial Node	Burnup DP5 to SP60			Burnup SP60 to SP61		
	SP60	T-Fuel	Spec.Vol	SP60	T-Fuel	Spec.Vol
1	9.667	1068.9	0.0250	12.060	977.9	0.0234
2	15.112	1199.7	0.0249	18.668	1091.3	0.0233
3	17.801	1235.2	0.0247	21.810	1111.7	0.0231
4	18.956	1233.3	0.0245	23.103	1104.3	0.0230
5	19.445	1223.0	0.0243	23.606	1090.0	0.0229
6	19.648	1213.5	0.0241	23.779	1075.4	0.0228
7	19.723	1206.8	0.0239	23.815	1063.8	0.0227
8	19.743	1202.3	0.0238	23.804	1057.0	0.0226
9	19.751	1198.7	0.0236	23.795	1055.9	0.0225
10	19.780	1195.4	0.0235	23.816	1058.5	0.0223
11	19.852	1192.7	0.0233	23.872	1061.9	0.0222
12	19.953	1190.9	0.0232	23.942	1063.3	0.0221
13	20.040	1190.2	0.0230	23.981	1061.6	0.0220
14	20.063	1190.1	0.0229	23.941	1056.5	0.0219
15	19.906	1188.8	0.0227	23.701	1048.6	0.0218
16	19.140	1181.9	0.0226	22.790	1036.6	0.0217
17	16.673	1148.5	0.0225	19.940	1006.9	0.0217
18	10.647	1013.2	0.0223	12.850	898.8	0.0216

Datapoint
or

Statepoint	EFPD / Cycle
DP2	0.0 / Cy3
DP3	142.8 / Cy3
DP4	0.0 / Cy4
DP5	126.6 / Cy4
SP60	0.0 / Cy5
SP61	114.4 / Cy5

Burnup	- GWd/MTU
T-Fuel	- °F
Spec. Vol.	- ft ³ / lbm

Table 5.2.8-8. Burnup and Thermal Hydraulic Parameters for Assembly C15a

Axial Node	Burnup DP2 to DP3			Burnup DP3 to DP4			Burnup DP4 to DP5		
	DP3	T-Fuel	Spec.Vol	DP4	T-Fuel	Spec.Vol	DP5	T-Fuel	Spec.Vol
1	1.575	863.7	0.0242	3.506	909.0	0.0242	6.192	1076.3	0.0252
2	2.714	1033.8	0.0241	5.815	1060.6	0.0242	10.051	1252.2	0.0250
3	3.412	1126.1	0.0240	7.108	1124.5	0.0240	12.138	1326.5	0.0248
4	3.795	1172.0	0.0239	7.772	1145.4	0.0239	13.153	1348.5	0.0246
5	3.997	1191.2	0.0238	8.130	1150.8	0.0238	13.646	1349.1	0.0244
6	4.098	1197.5	0.0236	8.330	1151.8	0.0237	13.880	1343.2	0.0242
7	4.143	1197.6	0.0235	8.443	1152.3	0.0236	13.980	1335.0	0.0240
8	4.155	1195.0	0.0234	8.510	1153.2	0.0234	14.016	1325.5	0.0238
9	4.151	1192.4	0.0233	8.557	1154.7	0.0233	14.037	1315.2	0.0236
10	4.153	1192.5	0.0232	8.608	1156.7	0.0232	14.079	1305.8	0.0235
11	4.178	1197.1	0.0231	8.682	1159.4	0.0231	14.165	1298.1	0.0233
12	4.232	1206.7	0.0230	8.785	1163.4	0.0230	14.276	1291.0	0.0231
13	4.313	1220.3	0.0228	8.912	1169.6	0.0229	14.363	1280.9	0.0230
14	4.406	1236.3	0.0227	9.058	1180.0	0.0228	14.387	1264.9	0.0228
15	4.471	1248.7	0.0226	9.158	1192.0	0.0226	14.258	1239.7	0.0227
16	4.342	1233.3	0.0225	8.909	1188.2	0.0225	13.627	1200.7	0.0225
17	3.673	1141.5	0.0224	7.662	1132.9	0.0224	11.680	1132.1	0.0224
18	2.164	920.3	0.0223	4.672	956.7	0.0223	7.229	975.1	0.0223

Axial Node	Burnup DP5 to SP60			Burnup SP60 to SP61		
	SP60	T-Fuel	Spec.Vol	SP60	T-Fuel	Spec.Vol
1	9.598	1065.8	0.0250	11.717	952.1	0.0233
2	15.067	1197.8	0.0249	18.318	1069.7	0.0232
3	17.797	1234.5	0.0247	21.603	1097.8	0.0231
4	18.976	1233.0	0.0245	23.013	1095.6	0.0230
5	19.478	1222.9	0.0243	23.572	1082.8	0.0228
6	19.690	1213.6	0.0241	23.739	1062.8	0.0227
7	19.774	1206.9	0.0240	23.689	1032.8	0.0226
8	19.801	1202.4	0.0238	23.522	1001.1	0.0225
9	19.810	1198.9	0.0236	23.379	991.0	0.0224
10	19.839	1196.0	0.0235	23.374	1001.1	0.0223
11	19.917	1194.0	0.0233	23.502	1021.1	0.0222
12	20.031	1193.1	0.0232	23.732	1049.1	0.0222
13	20.123	1192.9	0.0230	23.955	1073.3	0.0220
14	20.147	1193.0	0.0229	24.032	1075.7	0.0219
15	19.988	1191.5	0.0227	23.819	1063.0	0.0218
16	19.211	1184.1	0.0226	22.882	1044.7	0.0217
17	16.726	1150.2	0.0225	19.992	1010.0	0.0217
18	10.679	1014.4	0.0223	12.875	900.0	0.0216

Datapoint

or

Statepoint	EFPD / Cycle
DP2	0.0 / Cy3
DP3	142.8 / Cy3
DP4	0.0 / Cy4
DP5	126.6 / Cy4
SP60	0.0 / Cy5
SP61	114.4 / Cy5

Burnup	- GWd/MTU
T-Fuel	- °F
Spec. Vol.	- ft ³ / lbm

Table 5.2.8-9. Burnup and Thermal Hydraulic Parameters for Assembly C20

Axial Node	Burnup DP2 to DP3			Burnup DP3 to DP4			Burnup DP4 to DP5		
	DP3	T-Fuel	Spec.Vol	DP4	T-Fuel	Spec.Vol	DP5	T-Fuel	Spec.Vol
1	2.479	1018.8	0.0252	5.378	1054.9	0.0252	7.982	1043.9	0.0250
2	4.171	1248.0	0.0251	8.673	1228.4	0.0250	12.752	1198.4	0.0248
3	5.166	1367.3	0.0249	10.420	1297.0	0.0248	15.249	1265.0	0.0247
4	5.688	1422.1	0.0247	11.250	1317.4	0.0247	16.405	1283.3	0.0245
5	5.952	1443.3	0.0245	11.654	1321.0	0.0245	16.927	1283.4	0.0243
6	6.078	1448.2	0.0243	11.865	1320.7	0.0243	17.148	1277.0	0.0241
7	6.123	1445.0	0.0241	11.972	1320.7	0.0241	17.199	1267.1	0.0239
8	6.104	1436.6	0.0239	12.001	1320.9	0.0239	17.123	1252.4	0.0237
9	6.038	1426.0	0.0238	11.966	1320.5	0.0237	16.996	1235.1	0.0236
10	5.970	1418.7	0.0236	11.924	1319.8	0.0236	16.945	1223.3	0.0234
11	5.963	1421.4	0.0234	11.959	1321.3	0.0234	17.026	1216.4	0.0232
12	6.038	1434.7	0.0233	12.097	1325.7	0.0232	17.227	1211.1	0.0231
13	6.180	1455.9	0.0231	12.319	1333.1	0.0231	17.487	1206.8	0.0229
14	6.344	1477.2	0.0229	12.567	1342.8	0.0229	17.668	1199.4	0.0228
15	6.412	1486.7	0.0228	12.667	1349.9	0.0228	17.542	1178.4	0.0227
16	6.156	1459.0	0.0226	12.240	1342.1	0.0226	16.732	1143.6	0.0225
17	5.204	1341.2	0.0224	10.577	1280.7	0.0225	14.402	1082.2	0.0224
18	3.110	1062.3	0.0223	6.581	1080.1	0.0224	9.037	942.7	0.0223

Axial Node	Burnup DP5 to SP60			Burnup SP60 to SP61		
	SP60	T-Fuel	Spec.Vol	SP60	T-Fuel	Spec.Vol
1	11.333	1038.5	0.0249	13.590	963.6	0.0234
2	17.654	1160.8	0.0247	21.094	1073.0	0.0233
3	20.766	1188.1	0.0246	24.770	1099.1	0.0232
4	22.073	1183.6	0.0244	26.283	1093.8	0.0231
5	22.598	1173.6	0.0242	26.845	1079.5	0.0229
6	22.798	1164.9	0.0240	27.014	1063.5	0.0228
7	22.836	1158.9	0.0239	26.998	1048.9	0.0227
8	22.745	1153.9	0.0237	26.859	1040.2	0.0226
9	22.568	1147.5	0.0236	26.666	1041.2	0.0225
10	22.437	1140.2	0.0234	26.545	1049.2	0.0224
11	22.460	1135.0	0.0233	26.578	1058.3	0.0223
12	22.634	1131.9	0.0231	26.747	1064.6	0.0222
13	22.898	1131.3	0.0230	26.979	1064.6	0.0221
14	23.113	1133.7	0.0228	27.131	1057.4	0.0220
15	22.999	1135.3	0.0227	26.927	1047.2	0.0218
16	22.077	1130.8	0.0226	25.853	1034.1	0.0218
17	19.250	1105.8	0.0224	22.631	1002.4	0.0217
18	12.383	986.2	0.0223	14.671	902.3	0.0216

Datapoint
or

Statepoint	EFPD / Cycle
DP2	0.0 / Cy3
DP3	142.8 / Cy3
DP4	0.0 / Cy4
DP5	126.6 / Cy4
SP60	0.0 / Cy5
SP61	114.4 / Cy5

Burnup	- GWd/MTU
T-Fuel	- °F
Spec. Vol.	- ft ³ / lbm

Table 5.2.8-10. Burnup and Thermal Hydraulic Parameters for Assembly C21

Axial Node	Burnup DP2 to DP3			Burnup DP3 to DP4			Burnup DP4 to DP5		
	DP3	T-Fuel	Spec.Vol	DP4	T-Fuel	Spec.Vol	DP5	T-Fuel	Spec.Vol
1	1.352	823.2	0.0239	3.012	864.3	0.0239	5.665	1076.3	0.0248
2	2.352	973.5	0.0238	5.044	1002.5	0.0239	9.271	1261.5	0.0246
3	2.971	1057.7	0.0237	6.186	1060.8	0.0238	11.234	1341.9	0.0245
4	3.309	1098.6	0.0236	6.757	1081.3	0.0237	12.180	1369.6	0.0243
5	3.484	1116.1	0.0235	7.043	1086.8	0.0236	12.613	1373.8	0.0241
6	3.568	1121.5	0.0234	7.191	1087.8	0.0235	12.777	1369.7	0.0239
7	3.601	1120.9	0.0233	7.271	1088.0	0.0234	12.713	1360.1	0.0237
8	3.603	1117.4	0.0232	7.310	1088.4	0.0233	12.156	1322.5	0.0235
9	3.587	1113.5	0.0231	7.327	1089.1	0.0232	10.975	1142.0	0.0234
10	3.574	1111.8	0.0230	7.344	1090.1	0.0231	10.496	1048.7	0.0232
11	3.583	1114.6	0.0229	7.386	1091.8	0.0230	10.526	1033.7	0.0231
12	3.623	1122.4	0.0228	7.460	1094.7	0.0229	10.798	1032.8	0.0231
13	3.688	1133.8	0.0227	7.560	1099.3	0.0228	11.576	1063.1	0.0230
14	3.752	1145.1	0.0227	7.652	1105.1	0.0227	12.749	1255.4	0.0228
15	3.752	1147.1	0.0226	7.641	1108.4	0.0226	12.832	1274.7	0.0227
16	3.554	1120.9	0.0225	7.292	1096.7	0.0225	12.103	1237.4	0.0226
17	2.940	1033.8	0.0224	6.165	1041.5	0.0224	10.222	1159.9	0.0224
18	1.700	847.2	0.0223	3.696	883.7	0.0223	6.236	986.5	0.0223

Axial Node	Burnup DP5 to SP60			Burnup SP60 to SP61		
	SP60	T-Fuel	Spec.Vol	SP60	T-Fuel	Spec.Vol
1	9.126	1074.4	0.0247	11.368	980.5	0.0235
2	14.390	1212.5	0.0246	17.874	1112.1	0.0234
3	17.026	1253.5	0.0244	21.137	1144.5	0.0233
4	18.160	1254.5	0.0242	22.519	1141.7	0.0231
5	18.621	1246.0	0.0240	23.036	1127.5	0.0230
6	18.782	1238.2	0.0239	23.150	1107.1	0.0229
7	18.720	1233.3	0.0237	22.958	1077.4	0.0228
8	18.106	1225.9	0.0235	22.177	1049.9	0.0227
9	16.278	1153.4	0.0234	20.319	1069.1	0.0226
10	14.321	1025.7	0.0233	18.499	1117.9	0.0224
11	14.107	1004.4	0.0232	18.370	1147.4	0.0223
12	14.351	1001.1	0.0231	18.744	1181.1	0.0222
13	15.241	1010.7	0.0230	19.738	1198.4	0.0221
14	17.449	1123.4	0.0229	21.822	1156.0	0.0220
15	18.503	1206.2	0.0227	22.703	1118.2	0.0219
16	17.815	1215.3	0.0226	21.836	1098.0	0.0218
17	15.389	1177.9	0.0225	18.962	1059.8	0.0217
18	9.736	1032.2	0.0223	12.114	932.8	0.0216

Datapoint

or

Statepoint	EFPD / Cycle
DP2	0.0 / Cy3
DP3	142.8 / Cy3
DP4	0.0 / Cy4
DP5	126.6 / Cy4
SP60	0.0 / Cy5
SP61	114.4 / Cy5

Burnup	- GWd/MTU
T-Fuel	- °F
Spec. Vol.	- ft ³ / lbm

Table 5.2.8-11. Burnup and Thermal Hydraulic Parameters for Assembly C25

Axial Node	Burnup DP2 to DP3			Burnup DP3 to DP4			Burnup DP4 to DP5		
	DP3	T-Fuel	Spec.Vol	DP4	T-Fuel	Spec.Vol	DP5	T-Fuel	Spec.Vol
1	1.917	919.0	0.0243	4.210	963.1	0.0244	6.635	1033.5	0.0250
2	3.231	1109.4	0.0243	6.832	1122.7	0.0243	10.798	1213.4	0.0249
3	3.980	1203.9	0.0241	8.187	1182.6	0.0242	13.026	1297.9	0.0247
4	4.350	1244.2	0.0240	8.794	1199.6	0.0241	14.052	1328.5	0.0245
5	4.520	1257.9	0.0239	9.063	1202.0	0.0239	14.493	1336.7	0.0243
6	4.588	1259.0	0.0237	9.185	1200.9	0.0238	14.650	1335.1	0.0241
7	4.596	1253.5	0.0236	9.232	1199.8	0.0237	14.612	1325.8	0.0239
8	4.557	1243.8	0.0235	9.219	1199.0	0.0235	14.371	1301.2	0.0237
9	4.482	1232.1	0.0233	9.158	1197.6	0.0234	14.006	1257.6	0.0235
10	4.408	1223.4	0.0232	9.094	1196.3	0.0233	13.762	1222.9	0.0234
11	4.380	1222.7	0.0231	9.090	1196.8	0.0231	13.744	1207.3	0.0232
12	4.419	1231.3	0.0230	9.171	1200.1	0.0230	13.931	1204.0	0.0231
13	4.517	1247.7	0.0229	9.326	1206.6	0.0229	14.288	1215.7	0.0230
14	4.635	1266.3	0.0227	9.506	1215.5	0.0228	14.638	1239.6	0.0228
15	4.683	1274.4	0.0226	9.572	1221.9	0.0227	14.617	1235.8	0.0227
16	4.493	1249.6	0.0225	9.236	1212.3	0.0225	13.909	1199.6	0.0225
17	3.794	1155.4	0.0224	7.957	1155.3	0.0224	11.897	1124.9	0.0224
18	2.260	934.3	0.0223	4.910	977.3	0.0223	7.383	965.0	0.0223

Axial Node	Burnup DP5 to SP60			Burnup SP60 to SP61		
	SP60	T-Fuel	Spec.Vol	SP60	T-Fuel	Spec.Vol
1	9.861	1042.7	0.0249	12.191	969.0	0.0234
2	15.696	1179.9	0.0248	19.194	1080.6	0.0233
3	18.645	1221.3	0.0246	22.632	1104.6	0.0231
4	19.879	1223.0	0.0244	24.045	1102.2	0.0230
5	20.361	1215.7	0.0242	24.568	1090.2	0.0229
6	20.522	1209.0	0.0241	24.688	1072.4	0.0228
7	20.487	1204.5	0.0239	24.535	1045.8	0.0227
8	20.210	1198.3	0.0237	24.075	1015.7	0.0226
9	19.664	1181.4	0.0236	23.379	1010.9	0.0225
10	19.130	1158.2	0.0234	22.820	1025.0	0.0224
11	18.954	1145.8	0.0233	22.695	1045.1	0.0223
12	19.105	1142.4	0.0231	22.953	1071.8	0.0222
13	19.534	1148.0	0.0230	23.508	1093.8	0.0221
14	20.104	1167.2	0.0229	24.130	1091.4	0.0220
15	20.277	1184.2	0.0227	24.242	1076.2	0.0219
16	19.511	1184.5	0.0226	23.306	1057.5	0.0218
17	16.953	1150.3	0.0225	20.320	1021.0	0.0217
18	10.811	1011.2	0.0223	13.058	907.0	0.0216

Datapoint

or

Statepoint	EFPD / Cycle
DP2	0.0 / Cy3
DP3	142.8 / Cy3
DP4	0.0 / Cy4
DP5	126.6 / Cy4
SP60	0.0 / Cy5
SP61	114.4 / Cy5

Burnup	- GWd/MTU
T-Fuel	- °F
Spec. Vol.	- ft ³ / lbm

Table 5.2.8-12. Burnup and Thermal Hydraulic Parameters for Assembly C28

Axial Node	Burnup DP2 to DP3			Burnup DP3 to DP4			Burnup DP4 to DP5		
	DP3	T-Fuel	Spec.Vol	DP4	T-Fuel	Spec.Vol	DP5	T-Fuel	Spec.Vol
1	1.178	788.1	0.0234	2.653	832.7	0.0236	4.906	1036.1	0.0250
2	1.985	910.6	0.0234	4.336	954.0	0.0236	8.081	1216.3	0.0249
3	2.427	972.9	0.0233	5.184	1002.4	0.0235	9.716	1297.5	0.0247
4	2.629	997.7	0.0233	5.543	1016.5	0.0234	10.441	1329.2	0.0245
5	2.710	1004.5	0.0232	5.686	1017.9	0.0233	10.744	1339.4	0.0243
6	2.732	1003.1	0.0231	5.741	1016.2	0.0232	10.848	1340.2	0.0241
7	2.721	997.5	0.0230	5.751	1014.4	0.0232	10.834	1335.7	0.0240
8	2.685	989.5	0.0230	5.728	1012.8	0.0231	10.739	1325.1	0.0238
9	2.634	980.9	0.0229	5.685	1011.2	0.0230	10.644	1311.6	0.0236
10	2.586	974.4	0.0228	5.644	1010.1	0.0229	10.659	1306.7	0.0235
11	2.565	972.9	0.0228	5.635	1010.4	0.0228	10.805	1312.2	0.0233
12	2.580	977.5	0.0227	5.674	1013.2	0.0228	11.033	1321.4	0.0231
13	2.631	987.4	0.0226	5.759	1018.8	0.0227	11.239	1327.1	0.0230
14	2.700	999.6	0.0226	5.866	1026.8	0.0226	11.298	1322.4	0.0228
15	2.742	1007.1	0.0225	5.924	1033.4	0.0225	11.075	1293.8	0.0227
16	2.655	994.4	0.0224	5.746	1026.7	0.0225	10.386	1238.1	0.0225
17	2.261	934.8	0.0224	4.960	979.6	0.0224	8.771	1145.5	0.0224
18	1.352	792.5	0.0223	3.044	840.8	0.0223	5.379	962.6	0.0223

Axial Node	Burnup DP5 to SP60			Burnup SP60 to SP61		
	SP60	T-Fuel	Spec.Vol	SP60	T-Fuel	Spec.Vol
1	8.052	1053.0	0.0250	10.121	957.9	0.0234
2	12.832	1190.4	0.0248	16.077	1088.7	0.0233
3	15.148	1238.0	0.0247	19.016	1129.6	0.0231
4	16.084	1245.4	0.0245	20.234	1134.4	0.0230
5	16.436	1241.3	0.0243	20.681	1126.2	0.0229
6	16.554	1236.3	0.0241	20.780	1109.2	0.0228
7	16.557	1233.3	0.0240	20.657	1079.3	0.0227
8	16.475	1231.0	0.0238	20.353	1041.8	0.0226
9	16.360	1226.9	0.0236	20.030	1029.0	0.0225
10	16.340	1222.9	0.0235	19.938	1036.7	0.0224
11	16.504	1224.1	0.0233	20.139	1054.9	0.0223
12	16.796	1228.9	0.0232	20.560	1084.7	0.0222
13	17.065	1234.4	0.0230	21.005	1114.6	0.0221
14	17.180	1240.5	0.0229	21.231	1121.5	0.0220
15	16.945	1241.7	0.0227	20.968	1112.2	0.0219
16	16.047	1227.5	0.0226	19.892	1093.4	0.0218
17	13.777	1175.1	0.0224	17.150	1049.1	0.0217
18	8.703	1022.4	0.0223	10.912	916.0	0.0216

Datapoint

or

Statepoint	EFPD / Cycle
DP2	0.0 / Cy3
DP3	142.8 / Cy3
DP4	0.0 / Cy4
DP5	126.6 / Cy4
SP60	0.0 / Cy5
SP61	114.4 / Cy5

Burnup	- GWd/MTU
T-Fuel	- °F
Spec. Vol.	- ft ³ / lbm

Table 5.2.8-13. Burnup and Thermal Hydraulic Parameters for Assembly C29

Axial Node	Burnup DP2 to DP3			Burnup DP3 to DP4			Burnup DP4 to DP5		
	DP3	T-Fuel	Spec.Vol	DP4	T-Fuel	Spec.Vol	DP5	T-Fuel	Spec.Vol
1	1.133	779.1	0.0233	2.577	827.6	0.0235	4.385	952.6	0.0254
2	1.872	891.5	0.0233	4.148	944.4	0.0235	8.006	1281.8	0.0252
3	2.250	946.1	0.0232	4.899	989.6	0.0234	10.044	1397.1	0.0250
4	2.405	965.3	0.0232	5.199	1000.7	0.0233	10.886	1425.1	0.0248
5	2.453	968.0	0.0231	5.315	1000.1	0.0233	11.195	1430.4	0.0246
6	2.455	964.2	0.0230	5.355	997.0	0.0232	11.280	1428.0	0.0243
7	2.432	957.3	0.0230	5.351	994.2	0.0231	11.243	1421.3	0.0241
8	2.389	948.2	0.0229	5.319	991.8	0.0230	11.127	1410.2	0.0239
9	2.334	938.7	0.0228	5.268	989.5	0.0230	10.992	1395.9	0.0237
10	2.283	931.5	0.0228	5.222	987.9	0.0229	10.914	1383.3	0.0235
11	2.258	929.3	0.0227	5.207	988.0	0.0228	10.923	1373.5	0.0234
12	2.269	933.3	0.0227	5.241	990.9	0.0228	11.004	1367.1	0.0232
13	2.320	943.5	0.0226	5.329	997.8	0.0227	11.116	1362.5	0.0230
14	2.407	959.1	0.0226	5.477	1010.7	0.0226	11.191	1353.4	0.0229
15	2.514	977.1	0.0225	5.656	1027.2	0.0225	11.131	1328.1	0.0227
16	2.542	981.4	0.0224	5.662	1031.2	0.0225	10.696	1279.9	0.0226
17	2.250	935.5	0.0224	5.024	991.0	0.0224	9.264	1193.1	0.0224
18	1.387	798.5	0.0223	3.157	852.4	0.0223	5.808	1006.0	0.0223

Axial Node	Burnup DP5 to SP60			Burnup SP60 to SP61		
	SP60	T-Fuel	Spec.Vol	SP60	T-Fuel	Spec.Vol
1	6.905	1001.6	0.0252	7.875	759.1	0.0225
2	13.037	1227.0	0.0250	14.601	823.7	0.0224
3	15.979	1286.7	0.0248	17.897	850.1	0.0224
4	17.029	1290.0	0.0246	19.144	859.6	0.0223
5	17.371	1283.4	0.0244	19.579	860.8	0.0223
6	17.454	1276.7	0.0242	19.693	857.7	0.0222
7	17.419	1272.4	0.0241	19.655	852.9	0.0222
8	17.300	1269.1	0.0239	19.520	849.6	0.0221
9	17.129	1264.3	0.0237	19.336	850.1	0.0220
10	16.985	1258.3	0.0235	19.193	854.7	0.0220
11	16.938	1253.7	0.0234	19.159	861.3	0.0219
12	16.993	1251.7	0.0232	19.230	867.4	0.0219
13	17.115	1252.9	0.0230	19.357	870.2	0.0218
14	17.227	1256.6	0.0229	19.444	867.3	0.0218
15	17.169	1257.4	0.0227	19.308	856.9	0.0217
16	16.584	1246.6	0.0226	18.557	836.9	0.0217
17	14.560	1199.9	0.0225	16.205	799.8	0.0216
18	9.384	1045.2	0.0224	10.399	721.6	0.0216

Datapoint

or

Statepoint	EFPD / Cycle
DP2	0.0 / Cy3
DP3	142.8 / Cy3
DP4	0.0 / Cy4
DP5	126.6 / Cy4
SP60	0.0 / Cy5
SP61	114.4 / Cy5

Burnup	- GWd/MTU
T-Fuel	- °F
Spec. Vol.	- ft ³ / lbm

Table 5.2.8-14. Burnup and Thermal Hydraulic Parameters for Assembly D8

Axial Node	Burnup DP4 to DP5			Burnup DP5 to SP60			Burnup SP60 to SP61		
	DP5	T-Fuel	Spec.Vol	SP60	T-Fuel	Spec.Vol	SP61	T-Fuel	Spec.Vol
1	1.156	832.8	0.0240	2.954	888.9	0.0241	5.571	1093.3	0.0238
2	2.100	1005.2	0.0239	5.144	1049.9	0.0240	9.206	1274.2	0.0237
3	2.736	1101.5	0.0238	6.436	1118.3	0.0239	11.129	1327.0	0.0235
4	3.085	1146.5	0.0237	7.040	1138.7	0.0238	11.958	1329.0	0.0234
5	3.256	1165.8	0.0236	7.304	1142.1	0.0237	12.272	1314.0	0.0232
6	3.336	1172.9	0.0235	7.424	1141.5	0.0236	12.370	1295.3	0.0231
7	3.376	1175.1	0.0234	7.493	1140.8	0.0235	12.391	1278.3	0.0229
8	3.407	1175.9	0.0233	7.557	1141.6	0.0234	12.409	1267.3	0.0228
9	3.452	1177.7	0.0232	7.644	1143.8	0.0232	12.469	1265.3	0.0227
10	3.519	1181.3	0.0230	7.759	1147.1	0.0231	12.576	1271.0	0.0225
11	3.594	1185.6	0.0229	7.881	1150.9	0.0230	12.695	1280.0	0.0224
12	3.645	1187.3	0.0228	7.969	1154.4	0.0229	12.774	1288.4	0.0223
13	3.633	1182.3	0.0227	7.973	1156.8	0.0228	12.755	1293.1	0.0221
14	3.526	1166.7	0.0226	7.849	1156.9	0.0227	12.587	1292.8	0.0220
15	3.302	1135.7	0.0225	7.541	1151.1	0.0226	12.204	1287.7	0.0219
16	2.927	1080.4	0.0224	6.927	1129.3	0.0225	11.410	1270.0	0.0218
17	2.312	985.5	0.0224	5.709	1062.7	0.0224	9.657	1208.7	0.0217
18	1.316	813.3	0.0223	3.400	891.8	0.0223	5.935	1021.4	0.0216

Table 5.2.8-15. Burnup and Thermal Hydraulic Parameters for Assembly D15

Axial Node	Burnup DP4 to DP5			Burnup DP5 to SP60			Burnup SP60 to SP61		
	DP5	T-Fuel	Spec.Vol	SP60	T-Fuel	Spec.Vol	SP61	T-Fuel	Spec.Vol
1	1.216	841.6	0.0240	3.065	893.1	0.0241	5.252	1020.9	0.0235
2	2.150	1008.4	0.0239	5.198	1047.7	0.0240	8.671	1190.5	0.0234
3	2.758	1100.9	0.0238	6.431	1113.3	0.0239	10.539	1249.2	0.0233
4	3.097	1146.3	0.0237	7.026	1134.6	0.0238	11.410	1260.9	0.0232
5	3.268	1166.6	0.0236	7.294	1138.4	0.0237	11.776	1254.3	0.0231
6	3.350	1174.3	0.0235	7.416	1138.0	0.0236	11.900	1238.6	0.0229
7	3.387	1176.3	0.0234	7.483	1137.4	0.0235	11.900	1215.6	0.0228
8	3.411	1176.3	0.0233	7.538	1137.9	0.0233	11.851	1193.5	0.0227
9	3.445	1176.6	0.0232	7.607	1139.4	0.0232	11.850	1190.1	0.0226
10	3.499	1178.4	0.0230	7.698	1141.9	0.0231	11.953	1204.1	0.0225
11	3.563	1181.1	0.0229	7.798	1144.8	0.0230	12.109	1225.3	0.0223
12	3.610	1182.1	0.0228	7.876	1147.9	0.0229	12.260	1249.6	0.0222
13	3.602	1177.7	0.0227	7.888	1150.5	0.0228	12.333	1267.8	0.0221
14	3.509	1163.9	0.0226	7.786	1151.5	0.0227	12.220	1267.3	0.0220
15	3.299	1134.6	0.0225	7.505	1146.9	0.0226	11.828	1251.0	0.0219
16	2.931	1080.5	0.0224	6.909	1126.1	0.0225	10.991	1220.5	0.0218
17	2.314	985.3	0.0224	5.691	1059.3	0.0224	9.217	1152.7	0.0217
18	1.312	812.5	0.0223	3.377	888.9	0.0223	5.607	974.1	0.0216

Datapoint

or

Statepoint	EFPD / Cycle
DP4	0.0 / Cy4
DP5	126.6 / Cy4
SP60	0.0 / Cy5
SP61	114.4 / Cy5

Burnup	- GWd/MTU
T-Fuel	- °F
Spec. Vol.	- ft ³ / lbm

Table 5.2.8-16. Burnup and Thermal Hydraulic Parameters for Assembly D20

Axial Node	Burnup DP4 to DP5			Burnup DP5 to SP60			Burnup SP60 to SP61		
	DP5	T-Fuel	Spec.Vol	SP60	T-Fuel	Spec.Vol	SP61	T-Fuel	Spec.Vol
1	1.975	990.7	0.0249	4.762	1032.0	0.0249	7.197	1039.0	0.0236
2	3.358	1210.9	0.0248	7.746	1206.6	0.0248	11.487	1187.3	0.0235
3	4.194	1325.4	0.0246	9.350	1274.0	0.0246	13.655	1235.4	0.0233
4	4.642	1379.9	0.0245	10.095	1293.1	0.0244	14.604	1238.1	0.0232
5	4.866	1403.5	0.0243	10.426	1295.1	0.0243	14.984	1225.1	0.0231
6	4.965	1412.0	0.0241	10.572	1293.4	0.0241	15.100	1205.8	0.0229
7	4.989	1412.3	0.0239	10.632	1292.1	0.0239	15.082	1183.6	0.0228
8	4.970	1406.6	0.0237	10.641	1291.5	0.0238	14.996	1165.8	0.0227
9	4.953	1398.0	0.0236	10.632	1290.2	0.0236	14.923	1162.1	0.0226
10	4.979	1392.1	0.0234	10.656	1288.9	0.0235	14.936	1172.1	0.0224
11	5.054	1391.4	0.0233	10.742	1289.6	0.0233	15.050	1188.6	0.0223
12	5.144	1393.5	0.0231	10.865	1292.8	0.0231	15.216	1206.0	0.0222
13	5.194	1394.4	0.0229	10.965	1298.5	0.0230	15.346	1217.2	0.0221
14	5.138	1386.9	0.0228	10.956	1305.5	0.0228	15.323	1216.4	0.0220
15	4.903	1358.9	0.0227	10.704	1307.8	0.0227	14.995	1206.1	0.0219
16	4.419	1296.5	0.0225	9.994	1290.8	0.0226	14.103	1185.7	0.0218
17	3.548	1175.3	0.0224	8.384	1220.5	0.0224	12.001	1129.3	0.0217
18	2.053	941.1	0.0223	5.101	1023.5	0.0223	7.450	975.2	0.0216

Table 5.2.8-17. Burnup and Thermal Hydraulic Parameters for Assembly D20a

Axial Node	Burnup DP4 to DP5			Burnup DP5 to SP60			Burnup SP60 to SP61		
	DP5	T-Fuel	Spec.Vol	SP60	T-Fuel	Spec.Vol	SP61	T-Fuel	Spec.Vol
1	1.975	990.7	0.0249	4.762	1032.0	0.0249	5.899	861.0	0.0233
2	3.358	1210.9	0.0248	7.746	1206.6	0.0248	9.981	1055.3	0.0232
3	4.194	1325.4	0.0246	9.350	1274.0	0.0246	12.587	1115.1	0.0231
4	4.642	1379.9	0.0245	10.095	1293.1	0.0244	13.735	1130.4	0.0230
5	4.866	1403.5	0.0243	10.426	1295.1	0.0243	14.207	1129.7	0.0229
6	4.965	1412.0	0.0241	10.572	1293.4	0.0241	14.400	1123.2	0.0228
7	4.989	1412.3	0.0239	10.632	1292.1	0.0239	14.469	1116.1	0.0226
8	4.970	1406.6	0.0237	10.641	1291.5	0.0238	14.494	1114.8	0.0225
9	4.953	1398.0	0.0236	10.632	1290.2	0.0236	14.542	1124.2	0.0224
10	4.979	1392.1	0.0234	10.656	1288.9	0.0235	14.652	1141.3	0.0223
11	5.054	1391.4	0.0233	10.742	1289.6	0.0233	14.795	1156.1	0.0222
12	5.144	1393.5	0.0231	10.865	1292.8	0.0231	14.922	1163.4	0.0221
13	5.194	1394.4	0.0229	10.965	1298.5	0.0230	14.966	1160.5	0.0220
14	5.138	1386.9	0.0228	10.956	1305.5	0.0228	14.830	1145.7	0.0219
15	4.903	1358.9	0.0227	10.704	1307.8	0.0227	14.390	1122.4	0.0218
16	4.419	1296.5	0.0225	9.994	1290.8	0.0226	13.415	1090.1	0.0217
17	3.548	1175.3	0.0224	8.384	1220.5	0.0224	11.296	1028.0	0.0216
18	2.053	941.1	0.0223	5.101	1023.5	0.0223	6.929	887.8	0.0216

Datapoint or
 Statepoint EFPD / Cycle
 DP4 0.0 / Cy4
 DP5 126.6 / Cy4
 SP60 0.0 / Cy5
 SP61 114.4 / Cy5

Burnup - GWd/MTU
 T-Fuel - °F
 Spec. Vol. - ft³ / lbm

Table 5.2.8-18. Burnup and Thermal Hydraulic Parameters for Assembly D21

Axial Node	Burnup DP4 to DP5			Burnup DP5 to SP60			Burnup SP60 to SP61		
	DP5	T-Fuel	Spec.Vol	SP60	T-Fuel	Spec.Vol	SP61	T-Fuel	Spec.Vol
1	1.050	803.3	0.0237	2.619	845.2	0.0238	5.100	1073.3	0.0235
2	1.849	944.8	0.0236	4.432	980.0	0.0237	8.366	1265.1	0.0234
3	2.359	1026.0	0.0236	5.468	1037.9	0.0236	10.100	1332.8	0.0232
4	2.645	1065.8	0.0235	5.975	1057.6	0.0235	10.898	1344.2	0.0231
5	2.791	1083.9	0.0234	6.206	1062.1	0.0234	11.219	1333.1	0.0230
6	2.859	1090.8	0.0233	6.309	1062.1	0.0233	11.272	1308.3	0.0228
7	2.885	1092.1	0.0232	6.357	1061.5	0.0233	11.011	1242.7	0.0227
8	2.892	1090.4	0.0231	6.383	1061.3	0.0232	10.238	1024.1	0.0226
9	2.901	1088.0	0.0230	6.408	1061.5	0.0231	9.329	994.4	0.0225
10	2.925	1086.5	0.0229	6.445	1062.1	0.0230	9.175	1002.9	0.0224
11	2.962	1086.3	0.0228	6.497	1063.5	0.0229	9.288	1029.5	0.0224
12	2.994	1086.2	0.0227	6.547	1065.7	0.0228	9.782	1138.7	0.0223
13	2.995	1083.3	0.0227	6.566	1068.4	0.0227	10.620	1324.7	0.0222
14	2.933	1073.6	0.0226	6.510	1070.4	0.0226	11.238	1349.4	0.0220
15	2.776	1050.9	0.0225	6.308	1067.3	0.0225	11.103	1338.2	0.0219
16	2.474	1006.3	0.0224	5.821	1048.1	0.0225	10.406	1309.4	0.0218
17	1.946	921.8	0.0224	4.774	988.2	0.0224	8.758	1233.0	0.0217
18	1.088	772.8	0.0223	2.794	836.5	0.0223	5.311	1027.9	0.0216

Table 5.2.8-19. Burnup and Thermal Hydraulic Parameters for Assembly D25

Axial Node	Burnup DP4 to DP5			Burnup DP5 to SP60			Burnup SP60 to SP61		
	DP5	T-Fuel	Spec.Vol	SP60	T-Fuel	Spec.Vol	SP61	T-Fuel	Spec.Vol
1	1.631	920.1	0.0244	3.945	960.0	0.0244	6.465	1069.5	0.0237
2	2.810	1117.1	0.0243	6.511	1125.1	0.0243	10.424	1237.0	0.0236
3	3.537	1221.5	0.0242	7.920	1188.5	0.0242	12.458	1287.8	0.0234
4	3.932	1271.0	0.0241	8.583	1206.9	0.0241	13.344	1288.1	0.0233
5	4.129	1292.3	0.0239	8.876	1209.3	0.0239	13.684	1273.1	0.0232
6	4.212	1299.5	0.0238	8.998	1207.6	0.0238	13.786	1255.8	0.0230
7	4.224	1298.6	0.0236	9.035	1206.1	0.0236	13.785	1241.7	0.0229
8	4.195	1291.8	0.0235	9.020	1204.8	0.0235	13.740	1234.3	0.0227
9	4.158	1281.7	0.0233	8.978	1202.8	0.0234	13.689	1235.4	0.0226
10	4.148	1272.8	0.0232	8.949	1200.5	0.0233	13.665	1242.9	0.0225
11	4.176	1268.0	0.0231	8.968	1199.8	0.0231	13.682	1251.6	0.0224
12	4.224	1266.9	0.0230	9.029	1201.5	0.0230	13.724	1257.5	0.0222
13	4.257	1266.6	0.0228	9.098	1206.0	0.0229	13.750	1258.2	0.0221
14	4.218	1260.9	0.0227	9.101	1212.0	0.0228	13.691	1253.8	0.0220
15	4.042	1239.1	0.0226	8.916	1213.8	0.0226	13.417	1245.6	0.0219
16	3.655	1187.5	0.0225	8.338	1198.0	0.0225	12.662	1228.0	0.0218
17	2.928	1081.8	0.0224	6.968	1134.0	0.0224	10.788	1171.9	0.0217
18	1.677	875.8	0.0223	4.187	950.1	0.0223	6.661	1002.6	0.0216

Datapoint or Statepoint	EFPD / Cycle
DP4	0.0 / Cy4
DP5	126.6 / Cy4
SP60	0.0 / Cy5
SP61	114.4 / Cy5

Burnup	- GWd/MTU
T-Fuel	- °F
Spec. Vol.	- ft ³ / lbm

Table 5.2.8-20. Burnup and Thermal Hydraulic Parameters for Assembly D25a

Axial Node	Burnup DP4 to DP5			Burnup DP5 to SP60			Burnup SP60 to SP61		
	DP5	T-Fuel	Spec.Vol	SP60	T-Fuel	Spec.Vol	SP61	T-Fuel	Spec.Vol
1	1.631	920.1	0.0244	3.947	960.0	0.0244	5.607	988.5	0.0238
2	2.810	1117.1	0.0243	6.515	1125.1	0.0243	9.666	1242.5	0.0237
3	3.537	1221.5	0.0242	7.924	1188.5	0.0242	12.346	1305.3	0.0235
4	3.932	1271.0	0.0241	8.589	1206.9	0.0241	13.433	1305.8	0.0234
5	4.129	1292.3	0.0239	8.882	1209.3	0.0239	13.811	1290.1	0.0232
6	4.212	1299.5	0.0238	9.004	1207.6	0.0238	13.907	1268.8	0.0231
7	4.224	1298.6	0.0236	9.041	1206.1	0.0236	13.869	1245.6	0.0229
8	4.195	1291.8	0.0235	9.027	1204.8	0.0235	13.780	1229.6	0.0228
9	4.158	1281.7	0.0233	8.985	1202.8	0.0234	13.722	1232.2	0.0227
10	4.148	1272.8	0.0232	8.958	1200.5	0.0233	13.738	1250.2	0.0225
11	4.176	1268.0	0.0231	8.977	1199.8	0.0231	13.813	1272.5	0.0224
12	4.224	1266.9	0.0230	9.039	1201.5	0.0230	13.922	1293.1	0.0223
13	4.257	1266.6	0.0228	9.106	1206.0	0.0229	13.998	1303.2	0.0222
14	4.218	1260.9	0.0227	9.109	1212.0	0.0228	13.939	1297.0	0.0220
15	4.042	1239.1	0.0226	8.922	1213.8	0.0226	13.640	1282.1	0.0219
16	3.655	1187.5	0.0225	8.343	1198.0	0.0225	12.860	1259.1	0.0218
17	2.928	1081.8	0.0224	6.972	1134.0	0.0224	10.952	1197.5	0.0217
18	1.677	875.8	0.0223	4.189	950.1	0.0223	6.761	1018.8	0.0216

Table 5.2.8-21. Burnup and Thermal Hydraulic Parameters for Assembly D27

Axial Node	Burnup DP4 to DP5			Burnup DP5 to SP60			Burnup SP60 to SP61		
	DP5	T-Fuel	Spec.Vol	SP60	T-Fuel	Spec.Vol	SP61	T-Fuel	Spec.Vol
1	1.827	966.1	0.0248	4.422	1008.2	0.0248	6.484	987.7	0.0236
2	3.222	1197.0	0.0247	7.445	1189.1	0.0247	10.801	1145.8	0.0235
3	4.130	1319.4	0.0246	9.156	1258.2	0.0245	13.216	1210.1	0.0234
4	4.620	1375.1	0.0244	9.941	1276.0	0.0244	14.349	1229.8	0.0232
5	4.856	1398.3	0.0242	10.277	1276.9	0.0242	14.836	1229.0	0.0231
6	4.950	1405.7	0.0240	10.413	1274.5	0.0240	15.003	1217.3	0.0230
7	4.957	1404.0	0.0239	10.449	1272.9	0.0239	14.983	1196.8	0.0228
8	4.913	1395.3	0.0237	10.423	1271.9	0.0237	14.846	1175.6	0.0227
9	4.858	1382.5	0.0235	10.364	1270.1	0.0236	14.688	1170.2	0.0226
10	4.834	1370.6	0.0234	10.316	1267.8	0.0234	14.618	1181.3	0.0225
11	4.851	1362.8	0.0232	10.314	1266.5	0.0233	14.665	1202.7	0.0224
12	4.896	1359.4	0.0231	10.360	1267.5	0.0231	14.808	1229.8	0.0222
13	4.930	1358.3	0.0229	10.426	1271.8	0.0230	14.974	1252.3	0.0221
14	4.891	1352.8	0.0228	10.434	1278.4	0.0228	15.018	1256.6	0.0220
15	4.709	1331.1	0.0226	10.255	1281.9	0.0227	14.745	1241.8	0.0219
16	4.297	1278.1	0.0225	9.665	1268.4	0.0226	13.872	1205.8	0.0218
17	3.495	1166.5	0.0224	8.189	1204.0	0.0224	11.753	1126.1	0.0217
18	2.044	938.9	0.0223	5.024	1013.6	0.0223	7.230	953.1	0.0216

Datapoint or

Statepoint	EFPD / Cycle
DP4	0.0 / Cy4
DP5	126.6 / Cy4
SP60	0.0 / Cy5
SP61	114.4 / Cy5

Burnup	- GWd/MTU
T-Fuel	- °F
Spec. Vol.	- ft ³ / lbm

Table 5.2.8-22. Burnup and Thermal Hydraulic Parameters for Assembly D28

Axial Node	Burnup DP4 to DP5			Burnup DP5 to SP60			Burnup SP60 to SP61		
	DP5	T-Fuel	Spec.Vol	SP60	T-Fuel	Spec.Vol	SP61	T-Fuel	Spec.Vol
1	1.097	812.1	0.0237	2.693	848.4	0.0238	5.253	1093.0	0.0238
2	1.933	959.1	0.0237	4.553	982.8	0.0237	8.587	1286.2	0.0237
3	2.469	1043.2	0.0236	5.612	1040.1	0.0236	10.341	1349.9	0.0236
4	2.767	1083.9	0.0235	6.122	1057.9	0.0235	11.118	1355.5	0.0234
5	2.915	1101.5	0.0234	6.346	1061.1	0.0234	11.410	1341.3	0.0232
6	2.976	1107.1	0.0233	6.436	1060.2	0.0233	11.468	1319.1	0.0231
7	2.985	1106.1	0.0232	6.461	1058.9	0.0232	11.397	1291.0	0.0229
8	2.966	1101.1	0.0231	6.451	1057.8	0.0232	11.263	1265.9	0.0228
9	2.942	1093.9	0.0230	6.425	1056.6	0.0231	11.154	1262.1	0.0227
10	2.930	1087.1	0.0229	6.405	1055.4	0.0230	11.133	1277.6	0.0226
11	2.938	1082.4	0.0228	6.405	1054.9	0.0229	11.184	1301.7	0.0224
12	2.956	1079.8	0.0227	6.426	1055.9	0.0228	11.283	1329.0	0.0223
13	2.963	1077.6	0.0227	6.449	1058.4	0.0227	11.375	1348.1	0.0222
14	2.924	1071.2	0.0226	6.427	1061.4	0.0226	11.357	1348.9	0.0220
15	2.797	1053.4	0.0225	6.279	1060.7	0.0225	11.128	1337.1	0.0219
16	2.525	1013.1	0.0224	5.853	1045.2	0.0225	10.492	1312.6	0.0218
17	2.010	931.2	0.0224	4.851	988.3	0.0224	8.911	1240.2	0.0217
18	1.136	780.8	0.0223	2.866	839.3	0.0223	5.453	1036.5	0.0216

Table 5.2.8-23. Burnup and Thermal Hydraulic Parameters for Assembly E8

Axial Node	Burnup SP60 to SP61		
	SP61	T-Fuel	Spec.Vol
1	1.028	832.7	0.0229
2	1.918	998.9	0.0229
3	2.536	1088.4	0.0228
4	2.882	1125.1	0.0227
5	3.045	1136.9	0.0226
6	3.118	1138.2	0.0225
7	3.155	1137.5	0.0224
8	3.194	1141.2	0.0224
9	3.259	1152.8	0.0223
10	3.339	1169.9	0.0222
11	3.393	1185.1	0.0221
12	3.400	1193.0	0.0220
13	3.349	1190.3	0.0219
14	3.230	1174.3	0.0218
15	3.040	1144.6	0.0218
16	2.743	1093.2	0.0217
17	2.212	997.1	0.0216
18	1.270	813.2	0.0216

Datapoint or

Statepoint	EFPD / Cycle
DP4	0.0 / Cy4
DP5	126.6 / Cy4
SP60	0.0 / Cy5
SP61	114.4 / Cy5

Burnup	- GWd/MTU
T-Fuel	- °F
Spec. Vol.	- ft ³ / lbm

Table 5.2.8-24. Burnup and Thermal Hydraulic Parameters for Assembly E15

Axial Node	Burnup SP60 to SP61		
	SP61	T-Fuel	Spec.Vol
1	1.062	828.4	0.0228
2	1.923	984.3	0.0228
3	2.476	1069.0	0.0227
4	2.786	1105.4	0.0227
5	2.939	1117.5	0.0226
6	3.008	1118.8	0.0225
7	3.041	1117.8	0.0224
8	3.076	1120.9	0.0223
9	3.138	1132.6	0.0222
10	3.218	1150.4	0.0222
11	3.274	1166.1	0.0221
12	3.286	1174.7	0.0220
13	3.241	1172.8	0.0219
14	3.126	1157.1	0.0218
15	2.937	1126.8	0.0218
16	2.641	1074.9	0.0217
17	2.120	980.3	0.0216
18	1.213	801.7	0.0216

Table 5.2.8-25. Burnup and Thermal Hydraulic Parameters for Assembly E20

Axial Node	Burnup SP60 to SP61		
	SP61	T-Fuel	Spec.Vol
1	1.767	979.8	0.0236
2	3.072	1186.6	0.0235
3	3.815	1292.1	0.0234
4	4.204	1335.1	0.0232
5	4.392	1347.0	0.0231
6	4.464	1343.8	0.0230
7	4.473	1334.3	0.0228
8	4.464	1328.4	0.0227
9	4.489	1336.7	0.0226
10	4.561	1358.5	0.0225
11	4.641	1384.2	0.0223
12	4.700	1406.5	0.0222
13	4.706	1416.2	0.0221
14	4.619	1405.9	0.0220
15	4.411	1373.4	0.0219
16	4.026	1310.4	0.0217
17	3.284	1187.0	0.0217
18	1.915	941.3	0.0216

Datapoint

or

Statepoint	EFPD / Cycle
SP60	0.0 / Cy5
SP61	114.4 / Cy5

Burnup	- GWd/MTU
T-Fuel	- °F
Spec. Vol.	- ft ³ / lbm

Table 5.2.8-26. Burnup and Thermal Hydraulic Parameters for Assembly E21

Axial Node	SP60 to SP61		
	Burnup SP61	T-Fuel	Spec.Vol
1	0.920	783.6	0.0226
2	1.661	914.9	0.0226
3	2.114	989.9	0.0225
4	2.364	1022.6	0.0225
5	2.490	1033.7	0.0224
6	2.546	1034.7	0.0223
7	2.568	1032.7	0.0223
8	2.585	1033.2	0.0222
9	2.617	1040.2	0.0221
10	2.664	1053.0	0.0221
11	2.705	1066.3	0.0220
12	2.722	1075.6	0.0219
13	2.701	1076.8	0.0219
14	2.625	1066.2	0.0218
15	2.481	1041.8	0.0217
16	2.228	996.9	0.0217
17	1.770	911.8	0.0216
18	0.994	757.6	0.0216

Table 5.2.8-27. Burnup and Thermal Hydraulic Parameters for Assembly E25

Axial Node	Burnup SP60 to SP61		
	SP61	T-Fuel	Spec.Vol
1	1.521	922.1	0.0233
2	2.697	1115.5	0.0232
3	3.389	1217.1	0.0231
4	3.760	1259.4	0.0230
5	3.941	1271.9	0.0229
6	4.010	1269.6	0.0228
7	4.015	1260.8	0.0227
8	3.992	1253.7	0.0226
9	3.979	1255.9	0.0225
10	4.000	1268.9	0.0224
11	4.047	1288.4	0.0223
12	4.101	1308.5	0.0221
13	4.131	1321.2	0.0220
14	4.096	1318.7	0.0219
15	3.949	1295.2	0.0218
16	3.610	1239.4	0.0217
17	2.917	1122.7	0.0216
18	1.668	891.6	0.0216

Datapoint
or

Statepoint	EFPD / Cycle
SP60	0.0 / Cy5
SP61	114.4 / Cy5

Burnup	- GWd/MTU
T-Fuel	- °F
Spec. Vol.	- ft ³ / lbm

Table 5.2.8-28. Burnup and Thermal Hydraulic Parameters for Assembly E27

Axial Node	Burnup SP60 to SP61		
	SP61	T-Fuel	Spec.Vol
1	1.684	980.5	0.0236
2	3.015	1198.3	0.0235
3	3.856	1308.9	0.0234
4	4.305	1351.1	0.0232
5	4.510	1361.4	0.0231
6	4.580	1356.0	0.0230
7	4.571	1343.5	0.0228
8	4.526	1332.6	0.0227
9	4.488	1332.2	0.0226
10	4.489	1344.5	0.0225
11	4.531	1365.8	0.0224
12	4.595	1389.4	0.0222
13	4.646	1405.4	0.0221
14	4.635	1406.5	0.0220
15	4.505	1386.8	0.0219
16	4.164	1331.7	0.0218
17	3.422	1209.1	0.0217
18	2.002	958.0	0.0216

Table 5.2.8-29. Burnup and Thermal Hydraulic Parameters for Assembly E28

Axial Node	Burnup SP60 to SP61		
	SP61	T-Fuel	Spec.Vol
1	1.024	808.6	0.0227
2	1.852	952.9	0.0227
3	2.360	1034.0	0.0226
4	2.640	1068.2	0.0225
5	2.777	1078.8	0.0225
6	2.829	1077.6	0.0224
7	2.834	1071.7	0.0223
8	2.818	1066.7	0.0222
9	2.804	1067.3	0.0222
10	2.808	1074.8	0.0221
11	2.830	1086.8	0.0220
12	2.860	1099.5	0.0220
13	2.877	1107.7	0.0219
14	2.854	1106.1	0.0218
15	2.753	1089.3	0.0217
16	2.509	1046.7	0.0217
17	2.011	955.4	0.0216
18	1.134	785.0	0.0216

Datapoint
or

Statepoint	EFPD / Cycle
SP60	0.0 / Cy5
SP61	114.4 / Cy5

Burnup	- GWd/MTU
T-Fuel	- °F
Spec. Vol.	- ft ³ / lbm

5.2.9. RCCA Insertion History Data for Three Mile Island Unit 1 Depletion Calculations

The RCCA insertion time, duration, and position were required to perform the fuel assembly depletion calculations in which an RCCA was inserted. Hardening (locally increasing the average energy of the neutron population due to less local thermalization and increased local capture of neutrons at thermal energies) the neutron spectrum in a particular axial region of an assembly at a time during its irradiation history affects the isotopic composition of the depleted fuel. The CRC depletion calculations for fuel assemblies with an RCCA insertion history required the knowledge of the RCCA insertion time in terms of the number of EFPDs inserted in each axial node for each statepoint depletion calculation. Tables 5.2.9-1 through 5.2.9-9 present the RCCA insertion time data required for the fuel assembly depletion calculations relevant to the Three Mile Island Unit 1 CRC evaluations. These tables are obtained from pages 61 through 65 of Reference 7.5. The heights corresponding to the axial nodes presented in Tables 5.2.9-1 through 5.2.9-9 are: 20.0660 cm for the top and bottom nodes, 20.0025 cm for the intermediate nodes (p. 39, Ref. 7.5). The top of node 1 begins at the top of the active fuel region.

Table 5.2.9-1. Rod Insertion Time by Axial Node for Assembly B15

Axial Node	Time Rod Inserted (EFPD)	
	DP2 to DP3	DP3 to DP4
1	0.0	0.0
2	0.0	0.0
3	0.0	0.0
4	0.0	0.0
5	0.0	0.0
6	0.0	0.0
7	0.9	0.0
8	1.0	0.0
9	15.6	26.7
10	141.4	131.0
11	142.3	144.7
12	139.3	144.7
13	137.2	141.9
14	64.7	60.2
15	0.0	0.3
16	0.0	0.0
17	0.0	0.0
18	0.0	0.0

Datapoint or Statepoint	EFPD / Cycle
DP2	0.0 / Cy3
DP3	142.8 / Cy3
DP4	0.0 / Cy4
DP5	126.6 / Cy4

Table 5.2.9-2. Rod Insertion Time by Axial Node for Assembly B21

Axial Node	Time Rod Inserted (EFPD)	
	DP4 to DP5	DP5 to SP60
1	124.9	124.3
2	23.1	0.7
3	3.5	0.0
4	0.0	0.0
5	0.0	0.0
6	0.0	0.0
7	0.0	0.0
8	0.0	0.0
9	0.0	0.0
10	0.0	0.0
11	0.0	0.0
12	0.0	0.0
13	0.0	0.0
14	0.0	0.0
15	0.0	0.0
16	0.0	0.0
17	0.0	0.0
18	0.0	0.0

Table 5.2.9-3. Rod Insertion Time by Axial Node for Assembly B21a

Axial Node	Time Rod Inserted (EFPD)
	SP60 to SP61
1	103.3
2	46.6
3	3.0
4	0.0
5	0.0
6	0.0
7	0.0
8	0.0
9	0.0
10	0.0
11	0.0
12	0.0
13	0.0
14	0.0
15	0.0
16	0.0
17	0.0
18	0.0

Datapoint or Statepoint	EFPD / Cycle
DP4	0.0 / Cy4
DP5	126.6 / Cy4
SP60	0.0 / Cy5
SP61	114.4 / Cy5

Table 5.2.9-4. Rod Insertion Time by Axial Node for Assembly B29

Axial Node	Time Rod Inserted (EFPD)	
	DP4 to DP5	DP5 to SP60
1	124.9	124.3
2	23.0	0.7
3	3.4	0.0
4	0.0	0.0
5	0.0	0.0
6	0.0	0.0
7	0.0	0.0
8	0.0	0.0
9	0.0	0.0
10	0.0	0.0
11	0.0	0.0
12	0.0	0.0
13	0.0	0.0
14	0.0	0.0
15	0.0	0.0
16	0.0	0.0
17	0.0	0.0
18	0.0	0.0

Table 5.2.9-5. Rod Insertion Time by Axial Node for Assembly C21

Axial Node	Time Rod Inserted (EFPD)	
	DP4 to DP5	DP5 to SP60
1	0.0	0.0
2	0.0	0.0
3	0.0	0.0
4	0.0	0.0
5	0.0	0.0
6	0.0	0.0
7	0.4	0.0
8	16.3	0.0
9	92.2	28.7
10	126.6	146.5
11	126.6	147.4
12	118.3	147.4
13	72.3	147.4
14	5.1	52.6
15	0.0	0.0
16	0.0	0.0
17	0.0	0.0
18	0.0	0.0

Datapoint or Statepoint	EFPD / Cycle
DP4	0.0 / Cy4
DP5	126.6 / Cy4
SP60	0.0 / Cy5
SP61	114.4 / Cy5

Table 5.2.9-6. Rod Insertion Time by Axial Node for Assembly C29

Axial Node	Time Rod Inserted (EFPD)	
	DP4 to DP5	DP5 to SP60
1	124.9	124.3
2	23.3	0.7
3	3.6	0.0
4	0.0	0.0
5	0.0	0.0
6	0.0	0.0
7	0.0	0.0
8	0.0	0.0
9	0.0	0.0
10	0.0	0.0
11	0.0	0.0
12	0.0	0.0
13	0.0	0.0
14	0.0	0.0
15	0.0	0.0
16	0.0	0.0
17	0.0	0.0
18	0.0	0.0

Table 5.2.9-7. Rod Insertion Time by Axial Node for Assembly D20a

Axial Node	Time Rod Inserted (EFPD)
	SP60 to SP61
1	103.1
2	46.1
3	2.9
4	0.0
5	0.0
6	0.0
7	0.0
8	0.0
9	0.0
10	0.0
11	0.0
12	0.0
13	0.0
14	0.0
15	0.0
16	0.0
17	0.0
18	0.0

Datapoint or Statepoint	EFPD / Cycle
SP60	0.0 / Cy5
SP61	114.4 / Cy5

Table 5.2.9-8. Rod Insertion Time by Axial Node for Assembly D21

Axial Node	Time Rod Inserted (EFPD) SP60 to SP61
1	0.0
2	0.0
3	0.0
4	0.0
5	0.0
6	0.0
7	5.1
8	36.6
9	105.2
10	114.4
11	114.4
12	82.2
13	32.5
14	1.6
15	0.0
16	0.0
17	0.0
18	0.0

Table 5.2.9-9. Rod Insertion Time by Axial Node for Assembly D25a

Axial Node	Time Rod Inserted (EFPD) SP60 to SP61
1	103.4
2	46.7
3	3.0
4	0.0
5	0.0
6	0.0
7	0.0
8	0.0
9	0.0
10	0.0
11	0.0
12	0.0
13	0.0
14	0.0
15	0.0
16	0.0
17	0.0
18	0.0

Datapoint or Statepoint

Statepoint	EFPD / Cycle
SP60	0.0 / Cy5
SP61	114.4 / Cy5

5.3. Assembly Depletion Calculation Procedure

The procedure for performing the fuel assembly SAS2H depletion calculations documented in this analysis was based on the utilization of the CRAFT, Versions 4 and 5, software routine. The CRAFT software routine is described generally in Sections 5.6 and 5.7. The complete detailed description of the CRAFT, Versions 4 and 5, software routine is provided in Attachment I of Reference 7.6. The procedure for performing a fuel assembly depletion calculation with CRAFT, Versions 4 and 5, consisted of the following steps:

- Create a CRAFT input deck for the assembly depletion calculation.
- Assure that the CRAFT executable file, the CRAFT input deck entitled "datain", and the "sedexecute" executable file are in the same directory. The "sedexecute" executable file is a script file which is used in conjunction with the CRAFT code to create the consolidated output files described in Section 5.7.
- Execute CRAFT.
- Check and analyze the CRAFT generated SAS2H input decks and the SAS2H isotopic results.

The various CRAFT generated and consolidated SAS2H output files contain unique filenames which specify the following information:

- reactor identifier
- one-eighth core symmetry assembly number in current reactor cycle
- axial node number
- reactor cycle number in which the SAS2H calculation begins
- EFPD statepoint at which the SAS2H calculation begins
- reactor cycle number in which the SAS2H calculation ends
- EFPD statepoint at which the SAS2H calculation ends.

A complete detailed description of the filename content and format is provided in Attachment I of Reference 7.6.

5.4. Path B Model Development for the Three Mile Island Unit 1 Depletion Calculations

The SAS2H control module used ORIGEN-S to perform a point depletion calculation for the fuel assembly or section of the fuel assembly described in the SAS2H input deck. The ORIGEN-S calculational module used cell-weighted cross sections based on one-dimensional (1-D) transport calculations performed by XSDRNPM. One-dimensional transport calculations were performed on two models, Path A and Path B, to calculate energy dependent spatial neutron flux distributions necessary to perform cross section cell-weighting calculations.

The Path A model was simply a unit cell of the fuel assembly lattice containing a fuel rod. In the Path A model, the fuel pellet, gap, and clad were modeled explicitly. The only modification required to develop the Path A model was the conversion of the fuel assembly's square lattice unit cell perimeter to a radial perimeter conserving moderator volume within the unit cell (exterior to the fuel rod cladding). This

modification was performed automatically by the SAS2H control module. A 1-D transport calculation was performed on the Path A model for each energy group, and the spatial flux distributions for each energy group were used to calculate cell-weighted cross sections for the fuel.

The Path B model was a larger representation of the assembly than the Path A model. The Path B model approximated spectral effects due to heterogeneity within the fuel assembly such as water gaps, burnable poison rods, control rods, or axial power shaping rods. Typically, fuel assemblies contain a number of similar non-fuel lattice cells dispersed somewhat uniformly throughout the assembly lattice. The structure of the Path B model was based on a uniform distribution of these non-fuel lattice cells. In reality, most fuel assemblies do not have uniformly distributed non-fuel lattice cells, but the approximation of uniformly distributed non-fuel lattice cells was considered acceptable within the fidelity of these calculations as documented in Section S2.2.3.1 of Volume 1, Rev. 5 in Reference 7.1.

The basic structure of the Path B model for the fuel assembly depletion calculations performed in this analysis included an inner region composed of a representation of the non-fuel assembly lattice cell. A region containing the homogenization of the Path A model surrounded the inner region in the Path B model. A final region representing the moderator in the assembly-to-assembly spacing surrounded the homogenized region in the Path B model. The size of each radial region that surrounded the inner region in the Path B model was determined by conserving both the fuel-to-moderator mass ratio and the fuel-to-absorber (either burnable poison or RCCA poison) mass ratio in the corresponding section of the fuel assembly. The cell-weighted cross sections from the Path A model were applied to the homogenized region during the Path B model transport calculations. New cell-weighted cross sections for each energy group were then developed using the unit cell spatial flux distribution results from the Path B model transport calculations. These cell-weighted cross sections were ultimately used in point depletion calculations performed by ORIGEN-S to calculate both the depleted fuel and the depleted burnable poison (if present) isotopic compositions in the corresponding section of the fuel assembly. A detailed description of the calculations used to produce time-dependent cross sections by SAS2H is documented in Section S2.2.4 of Volume 1, Rev. 5 in Reference 7.1.

The Path B models for the various fuel assembly configurations had to be provided to the SAS2H control module. The primary concern in the development of the Path B model for PWR assemblies was the conservation of the fuel-to-moderator and the fuel-to-absorber mass ratios in the corresponding section of the assembly.

The Path B model development calculations for the Three Mile Island Unit 1 depletion calculations are presented in Tables 5.4-1 through 5.4-8 and contain the following information:

- the fuel assembly section characteristics for which the Path B model is developed
- the required Path B model development input parameters
- the parameters calculated to determine the final Path B model dimensions
- references to equations from Table 5.4-9 that were used to calculate parameters
- the final Path B model dimension results.

Table 5.4-9 contains a listing of the equations referenced and utilized in each of the Path B model development calculations presented in Tables 5.4-1 through 5.4-8.

**Table 5.4-1. SAS2H Path B Model Dimension Calculations for Three Mile Island Unit 1:
No Insertion Assembly, Batch 4**

Input Parameters

Number of unit cells in assembly: 225
Number of fuel rods in assembly: 208
Number of guide tubes in assembly: 16
Rod pitch in assembly (cm): 1.44272
Fuel pellet diameter (cm): 0.9398
Fuel cladding outer diameter (cm): 1.0922
Guide tube outer diameter (cm): 1.3462
Guide tube inner diameter (cm): 1.26492
Instrument tube outer diameter (cm): 1.38193
Instrument tube inner diameter (cm): 1.12014
Assembly pitch (cm): 21.81098

Fuel-to-Moderator Unit Volume Ratio Calculation

Identifier of Equation Utilized: 5.4-1
Fuel-to-Moderator Unit Volume Ratio = 0.53386

Moderator Unit Volume in Central Region of Path B Model

Identifier of Equation Utilized: 5.4-2
Moderator Unit Volume in Central Region of Path B Model = 1.91476

Fuel Unit Volume in Fuel Rod Unit Cell (Path A Model)

Identifier of Equation Utilized: 5.4-3
Fuel Unit Volume in Fuel Rod Unit Cell = 0.69368

Moderator Unit Volume in Fuel Rod Unit Cell (Path A Model)

Identifier of Equation Utilized: 5.4-4
Moderator Unit Volume in Fuel Rod Unit Cell = 1.14454

**Number of Fuel Rod Unit Cells that must be Represented
in the Homogenized Region of the Path B Model**

Identifier of Equation Utilized: 5.4-6
Number of Fuel Rod Unit Cells that must be Represented in
the Homogenized Region of the Path B Model = 12.3674

Path B Model Dimensions

	<u>Region #</u>	<u>Outer Radius (cm)</u>	<u>Region Description</u>
Inner	1	0.63246	Water filled gap
	2	0.67310	Guide tube
	3	0.81397	Guide tube unit cell moderator
	4	2.97599	Homogenized region
Outer	5	2.99939	Moderator in the inter-assembly spacing

Notes: The Region 4 outer radius is calculated using Equation 5.4-7.
 The Region 5 outer radius is calculated using Equation 5.4-8.

Table 5.4-2. SAS2H Path B Model Dimension Calculations for Three Mile Island Unit 1: No Insertion Assembly, Batches 5, 6, 7

Input Parameters

Number of unit cells in assembly: 225
 Number of fuel rods in assembly: 208
 Number of guide tubes in assembly: 16
 Rod pitch in assembly (cm): 1.44272
 Fuel pellet diameter (cm): 0.93853
 Fuel cladding outer diameter (cm): 1.0922
 Guide tube outer diameter (cm): 1.3462
 Guide tube inner diameter (cm): 1.26492
 Instrument tube outer diameter (cm): 1.38193
 Instrument tube inner diameter (cm): 1.12014
 Assembly pitch (cm): 21.81098

Fuel-to-Moderator Unit Volume Ratio Calculation

Identifier of Equation Utilized: 5.4-1
 Fuel-to-Moderator Unit Volume Ratio = 0.53242

Moderator Unit Volume in Central Region of Path B Model

Identifier of Equation Utilized: 5.4-2
 Moderator Unit Volume in Central Region of Path B Model = 1.91476

Fuel Unit Volume in Fuel Rod Unit Cell (Path A Model)

Identifier of Equation Utilized: 5.4-3
 Fuel Unit Volume in Fuel Rod Unit Cell = 0.69181

Moderator Unit Volume in Fuel Rod Unit Cell (Path A Model)

Identifier of Equation Utilized: 5.4-4
 Moderator Unit Volume in Fuel Rod Unit Cell = 1.14454

Title: CRC Depletion Calculations for Three Mile Island Unit 1
Document Identifier: B00000000-01717-0210-00007 REV 00

**Number of Fuel Rod Unit Cells that must be Represented
in the Homogenized Region of the Path B Model**

Identifier of Equation Utilized: 5.4-6
Number of Fuel Rod Unit Cells that must be Represented in
the Homogenized Region of the Path B Model = 12.3674

Path B Model Dimensions

	Region #	Outer Radius (cm)	Region Description
Inner	1	0.63246	Water filled gap
	2	0.67310	Guide tube
	3	0.81397	Guide tube unit cell moderator
	4	2.97599	Homogenized region
Outer	5	2.99939	Moderator in the inter-assembly spacing

Notes: The Region 4 outer radius is calculated using Equation 5.4-7.
The Region 5 outer radius is calculated using Equation 5.4-8.

**Table 5.4-3. SAS2H Path B Model Dimension Calculations for Three Mile Island Unit 1:
RCCA Insertion, Batch 4**

Input Parameters

Number of unit cells in assembly: 225
Number of fuel rods in assembly: 208
Number of guide tubes in assembly: 16
Rod pitch in assembly (cm): 1.44272
Fuel pellet diameter (cm): 0.9398
Fuel cladding outer diameter (cm): 1.0922
Guide tube outer diameter (cm): 1.3462
Guide tube inner diameter (cm): 1.26492
CR cladding outer diameter (cm): 1.11760
CR cladding inner diameter (cm): 1.01092
CR absorber material diameter (cm): 0.99568
Instrument tube outer diameter (cm): 1.38193
Instrument tube inner diameter (cm): 1.12014
Assembly pitch (cm): 21.81098

Fuel-to-Moderator Unit Volume Ratio Calculation

Identifier of Equation Utilized: 5.4-1
Fuel-to-Moderator Unit Volume Ratio = 0.56678

Moderator Unit Volume in Central Region of Path B Model

Identifier of Equation Utilized: 5.4-2
Moderator Unit Volume in Central Region of Path B Model = 0.93377

Title: CRC Depletion Calculations for Three Mile Island Unit 1
Document Identifier: B00000000-01717-0210-00007 REV 00

Fuel Unit Volume in Fuel Rod Unit Cell (Path A Model)

Identifier of Equation Utilized: 5.4-3
Fuel Unit Volume in Fuel Rod Unit Cell = 0.69368

Moderator Unit Volume in Fuel Rod Unit Cell (Path A Model)

Identifier of Equation Utilized: 5.4-4
Moderator Unit Volume in Fuel Rod Unit Cell = 1.14454

Number of Fuel Rod Unit Cells that must be Represented in the Homogenized Region of the Path B Model

Identifier of Equation Utilized: 5.4-6
Number of Fuel Rod Unit Cells that must be Represented in the Homogenized Region of the Path B Model = 11.76595

Required Area of Neutron Absorber Material in Path B Model (cm²)

Identifier of Equation Utilized: 5.4-10
Neutron Absorber Area = 0.70471

Path B Model Dimensions

	<u>Region #</u>	<u>Outer Radius (cm)</u>	<u>Region Description</u>
Inner	1	0.47362	Neutron absorber material
	2	0.50546	Gap
	3	0.55880	Control rod cladding
	4	0.63246	Water filled gap
	5	0.67310	Guide tube
	6	0.81397	Guide tube unit cell moderator
	7	2.90826	Homogenized region
Outer	8	2.93113	Moderator in the inter-assembly spacing

Notes: The Region 7 outer radius is calculated using Equation 5.4-7.
The Region 8 outer radius is calculated using Equation 5.4-8.
The Region 1 radius is calculated using Equation 5.4-11.

**Table 5.4-4. SAS2H Path B Model Dimension Calculations for Three Mile Island Unit 1:
RCCA Insertion, Batches 5, 6, 7****Input Parameters**

Number of unit cells in assembly: 225
Number of fuel rods in assembly: 208
Number of guide tubes in assembly: 16
Rod pitch in assembly (cm): 1.44272
Fuel pellet diameter (cm): 0.93853
Fuel cladding outer diameter (cm): 1.0922
Guide tube outer diameter (cm): 1.3462
Guide tube inner diameter (cm): 1.26492
CR cladding outer diameter (cm): 1.11760
CR cladding inner diameter (cm): 1.01092
CR absorber material diameter (cm): 0.99568
Instrument tube outer diameter (cm): 1.38193
Instrument tube inner diameter (cm): 1.12014
Assembly pitch (cm): 21.81098

Fuel-to-Moderator Unit Volume Ratio Calculation

Identifier of Equation Utilized: 5.4-1
Fuel-to-Moderator Unit Volume Ratio = 0.56525

Moderator Unit Volume in Central Region of Path B Model

Identifier of Equation Utilized: 5.4-2
Moderator Unit Volume in Central Region of Path B Model = 0.93377

Fuel Unit Volume in Fuel Rod Unit Cell (Path A Model)

Identifier of Equation Utilized: 5.4-3
Fuel Unit Volume in Fuel Rod Unit Cell = 0.69181

Moderator Unit Volume in Fuel Rod Unit Cell (Path A Model)

Identifier of Equation Utilized: 5.4-4
Moderator Unit Volume in Fuel Rod Unit Cell = 1.14454

**Number of Fuel Rod Unit Cells that must be Represented
in the Homogenized Region of the Path B Model**

Identifier of Equation Utilized: 5.4-6
Number of Fuel Rod Unit Cells that must be Represented in
the Homogenized Region of the Path B Model = 11.76595

Required Area of Neutron Absorber Material in Path B Model (cm²)

Identifier of Equation Utilized: 5.4-10
Neutron Absorber Area = 0.704714

Path B Model Dimensions

	<u>Region #</u>	<u>Outer Radius (cm)</u>	<u>Region Description</u>
Inner	1	0.47362	Neutron absorber material
	2	0.50546	Gap
	3	0.55880	Control rod cladding
	4	0.63246	Water filled gap
	5	0.67310	Guide tube
	6	0.81397	Guide tube unit cell moderator
	7	2.90826	Homogenized region
Outer	8	2.93113	Moderator in the inter-assembly spacing

Notes: The Region 7 outer radius is calculated using Equation 5.4-7.
 The Region 8 outer radius is calculated using Equation 5.4-8.
 The Region 1 radius is calculated using Equation 5.4-11.

Table 5.4-5. SAS2H Path B Model Dimension Calculations for Three Mile Island Unit 1: APSRA Insertion, Absorber Region, Batch 4

Input Parameters

Number of unit cells in assembly: 225
 Number of fuel rods in assembly: 208
 Number of guide tubes in assembly: 16
 Rod pitch in assembly (cm): 1.44272
 Fuel pellet diameter (cm): 0.9398
 Fuel cladding outer diameter (cm): 1.0922
 Guide tube outer diameter (cm): 1.3462
 Guide tube inner diameter (cm): 1.26492
 APSR cladding outer diameter (cm): 1.11760
 APSR cladding inner diameter (cm): 1.01092
 APSR absorber material diameter (cm): 0.99568
 Instrument tube outer diameter (cm): 1.38193
 Instrument tube inner diameter (cm): 1.12014
 Assembly pitch (cm): 21.81098

Fuel-to-Moderator Unit Volume Ratio Calculation

Identifier of Equation Utilized: 5.4-1
 Fuel-to-Moderator Unit Volume Ratio = 0.56678

Moderator Unit Volume in Central Region of Path B Model

Identifier of Equation Utilized: 5.4-2
 Moderator Unit Volume in Central Region of Path B Model = 0.93377

Fuel Unit Volume in Fuel Rod Unit Cell (Path A Model)

Identifier of Equation Utilized: 5.4-3
 Fuel Unit Volume in Fuel Rod Unit Cell = 0.69368

Title: CRC Depletion Calculations for Three Mile Island Unit 1
Document Identifier: B00000000-01717-0210-00007 REV 00

Moderator Unit Volume in Fuel Rod Unit Cell (Path A Model)

Identifier of Equation Utilized: 5.4-4
 Moderator Unit Volume in Fuel Rod Unit Cell = 1.14454

Number of Fuel Rod Unit Cells that must be Represented in the Homogenized Region of the Path B Model

Identifier of Equation Utilized: 5.4-6
 Number of Fuel Rod Unit Cells that must be Represented in the Homogenized Region of the Path B Model = 11.76595

Required Area of Neutron Absorber Material in Path B Model (cm²)

Identifier of Equation Utilized: 5.4-10
 Neutron Absorber Area = 0.70471

Path B Model Dimensions

	<u>Region #</u>	<u>Outer Radius (cm)</u>	<u>Region Description</u>
Inner	1	0.47362	Neutron absorber material
	2	0.50546	Gap
	3	0.55880	APSR cladding
	4	0.63246	Water filled gap
	5	0.67310	Guide tube
	6	0.81397	Guide tube unit cell moderator
	7	2.90826	Homogenized region
Outer	8	2.93113	Moderator in the inter-assembly spacing

Notes: The Region 7 outer radius is calculated using Equation 5.4-7.
 The Region 8 outer radius is calculated using Equation 5.4-8.
 The Region 1 radius is calculated using Equation 5.4-11.

Table 5.4-6. SAS2H Path B Model Dimension Calculations for Three Mile Island Unit 1: APSRA Insertion, Follow Rod Region, Batch 4

Input Parameters

Number of unit cells in assembly: 225
 Number of fuel rods in assembly: 208
 Number of guide tubes in assembly: 16
 Rod pitch in assembly (cm): 1.44272
 Fuel pellet diameter (cm): 0.9398
 Fuel cladding outer diameter (cm): 1.0922
 Guide tube outer diameter (cm): 1.3462
 Guide tube inner diameter (cm): 1.26492
 APSR cladding outer diameter (cm): 1.11760
 APSR cladding inner diameter (cm): 1.01092
 APSR absorber material diameter (cm): 0.99568
 Instrument tube outer diameter (cm): 1.38193

Title: CRC Depletion Calculations for Three Mile Island Unit 1

Document Identifier: B00000000-01717-0210-00007 REV 00

Page 52 of 76

Instrument tube inner diameter (cm): 1.12014

Assembly pitch (cm): 21.81098

Fuel-to-Moderator Unit Volume Ratio Calculation

Identifier of Equation Utilized: 5.4-1

Fuel-to-Moderator Unit Volume Ratio = 0.53956

Moderator Unit Volume in Central Region of Path B Model

Identifier of Equation Utilized: 5.4-2

Moderator Unit Volume in Central Region of Path B Model = 1.73641

Fuel Unit Volume in Fuel Rod Unit Cell (Path A Model)

Identifier of Equation Utilized: 5.4-3

Fuel Unit Volume in Fuel Rod Unit Cell = 0.69368

Moderator Unit Volume in Fuel Rod Unit Cell (Path A Model)

Identifier of Equation Utilized: 5.4-4

Moderator Unit Volume in Fuel Rod Unit Cell = 1.14454

Number of Fuel Rod Unit Cells that must be Represented in the Homogenized Region of the Path B Model

Identifier of Equation Utilized: 5.4-6

Number of Fuel Rod Unit Cells that must be Represented in the Homogenized Region of the Path B Model = 12.30592

Path B Model Dimensions

	<u>Region #</u>	<u>Outer Radius (cm)</u>	<u>Region Description</u>
Inner	1	0.47362	Water
	2	0.50546	Water
	3	0.55880	APSR cladding
	4	0.63246	Water filled gap
	5	0.67310	Guide tube
	6	0.81397	Guide tube unit cell moderator
	7	2.96913	Homogenized region
Outer	8	2.99248	Moderator in the inter-assembly spacing

Notes: The Region 7 outer radius is calculated using Equation 5.4-7.

The Region 8 outer radius is calculated using Equation 5.4-8.

The Region 1 radius is calculated using Equation 5.4-11.

**Table 5.4-7. SAS2H Path B Model Dimension Calculations for Three Mile Island Unit 1:
APSR Insertion, Absorber Region, Batch 5, 6, 7****Input Parameters**

Number of unit cells in assembly: 225
Number of fuel rods in assembly: 208
Number of guide tubes in assembly: 16
Rod pitch in assembly (cm): 1.44272
Fuel pellet diameter (cm): 0.93853
Fuel cladding outer diameter (cm): 1.0922
Guide tube outer diameter (cm): 1.3462
Guide tube inner diameter (cm): 1.26492
APSR cladding outer diameter (cm): 1.11760
APSR cladding inner diameter (cm): 1.01092
APSR absorber material diameter (cm): 0.99568
Instrument tube outer diameter (cm): 1.38193
Instrument tube inner diameter (cm): 1.12014
Assembly pitch (cm): 21.81098

Fuel-to-Moderator Unit Volume Ratio Calculation

Identifier of Equation Utilized: 5.4-1
Fuel-to-Moderator Unit Volume Ratio = 0.56525

Moderator Unit Volume in Central Region of Path B Model

Identifier of Equation Utilized: 5.4-2
Moderator Unit Volume in Central Region of Path B Model = 0.93377

Fuel Unit Volume in Fuel Rod Unit Cell (Path A Model)

Identifier of Equation Utilized: 5.4-3
Fuel Unit Volume in Fuel Rod Unit Cell = 0.69181

Moderator Unit Volume in Fuel Rod Unit Cell (Path A Model)

Identifier of Equation Utilized: 5.4-4
Moderator Unit Volume in Fuel Rod Unit Cell = 1.14454

**Number of Fuel Rod Unit Cells that must be Represented
in the Homogenized Region of the Path B Model**

Identifier of Equation Utilized: 5.4-6
Number of Fuel Rod Unit Cells that must be Represented in
the Homogenized Region of the Path B Model = 11.76595

Required Area of Neutron Absorber Material in Path B Model (cm²)

Identifier of Equation Utilized: 5.4-10
Neutron Absorber Area = 0.70471

Path B Model Dimensions

	<u>Region #</u>	<u>Outer Radius (cm)</u>	<u>Region Description</u>
Inner	1	0.47362	Neutron absorber material
	2	0.50546	Gap
	3	0.55880	APSR cladding
	4	0.63246	Water filled gap
	5	0.67310	Guide tube
	6	0.81397	Guide tube unit cell moderator
	7	2.90826	Homogenized region
Outer	8	2.93113	Moderator in the inter-assembly spacing

Notes: The Region 7 outer radius is calculated using Equation 5.4-7.
 The Region 8 outer radius is calculated using Equation 5.4-8.
 The Region 1 radius is calculated using Equation 5.4-11.

Table 5.4-8. SAS2H Path B Model Dimension Calculations for Three Mile Island Unit 1: APSRA Insertion, Follow Rod Region, Batch 5, 6, 7

Input Parameters

Number of unit cells in assembly: 225
 Number of fuel rods in assembly: 208
 Number of guide tubes in assembly: 16
 Rod pitch in assembly (cm): 1.44272
 Fuel pellet diameter (cm): 0.93853
 Fuel cladding outer diameter (cm): 1.0922
 Guide tube outer diameter (cm): 1.3462
 Guide tube inner diameter (cm): 1.26492
 APSR cladding outer diameter (cm): 1.11760
 APSR cladding inner diameter (cm): 1.01092
 APSR absorber material diameter (cm): 0.99568
 Instrument tube outer diameter (cm): 1.38193
 Instrument tube inner diameter (cm): 1.12014
 Assembly pitch (cm): 21.81098

Fuel-to-Moderator Unit Volume Ratio Calculation

Identifier of Equation Utilized: 5.4-1
 Fuel-to-Moderator Unit Volume Ratio = 0.53810

Moderator Unit Volume in Central Region of Path B Model

Identifier of Equation Utilized: 5.4-2
 Moderator Unit Volume in Central Region of Path B Model = 1.73641

Fuel Unit Volume in Fuel Rod Unit Cell (Path A Model)

Identifier of Equation Utilized: 5.4-3
 Fuel Unit Volume in Fuel Rod Unit Cell = 0.69181

Title: CRC Depletion Calculations for Three Mile Island Unit 1
Document Identifier: B00000000-01717-0210-00007 REV 00

Page 55 of 76

Moderator Unit Volume in Fuel Rod Unit Cell (Path A Model)

Identifier of Equation Utilized: 5.4-4

Moderator Unit Volume in Fuel Rod Unit Cell = 1.14454

**Number of Fuel Rod Unit Cells that must be Represented
in the Homogenized Region of the Path B Model**

Identifier of Equation Utilized: 5.4-6

Number of Fuel Rod Unit Cells that must be Represented in
the Homogenized Region of the Path B Model = 12.30592

Path B Model Dimensions

	<u>Region #</u>	<u>Outer Radius (cm)</u>	<u>Region Description</u>
Inner	1	0.47362	Water
	2	0.50546	Water
	3	0.55880	APSR cladding
	4	0.63246	Water filled gap
	5	0.67310	Guide tube
	6	0.81397	Guide tube unit cell moderator
	7	2.96913	Homogenized region
Outer	8	2.99248	Moderator in the inter-assembly spacing

Notes: The Region 7 outer radius is calculated using Equation 5.4-7.
The Region 8 outer radius is calculated using Equation 5.4-8.
The Region 1 radius is calculated using Equation 5.4-11.

**Table 5.4-9. Equations Used in the Path B Model
Development for the Three Mile Island Unit 1 Depletion Calculations**

(The equations listed below were derived. All distance dimensions are in centimeters. All area dimensions are in square centimeters. All other parameters are dimensionless.)

Equation 5.4-1. Fuel-to-Moderator Unit Volume (Area) Ratio in Fuel Assembly Section

$$\frac{F}{M} \text{ Ratio} = \frac{(\# FR) \left(\frac{\pi}{4} \right) (FP \text{ Diameter})^2}{\left\{ (\# FR) \left[P^2 - (FR \text{ Clad OD})^2 \left(\frac{\pi}{4} \right) \right] + (\# \text{ Empty GTs}) \left[P^2 - (GT \text{ OD})^2 \left(\frac{\pi}{4} \right) + (GT \text{ ID})^2 \left(\frac{\pi}{4} \right) \right] + \right. \\ \left. (\# \text{ Rodded GTs}) \left[\begin{array}{l} P^2 - (GT \text{ OD})^2 \left(\frac{\pi}{4} \right) + (GT \text{ ID})^2 \left(\frac{\pi}{4} \right) - \\ (Inserted \text{ Rod Outer Clad OD})^2 \left(\frac{\pi}{4} \right) + \\ (Inserted \text{ Rod Inner Clad ID})^2 \left(\frac{\pi}{4} \right) \end{array} \right] + \left[P^2 - (IT \text{ OD})^2 \left(\frac{\pi}{4} \right) + (IT \text{ ID})^2 \left(\frac{\pi}{4} \right) \right] \right\}}$$

where: F=Fuel, M=Moderator, #=Number, FR=Fuel Rod, FP=Fuel Pellet, P=Cell Pitch, OD=Outer Diameter, GT=Guide Tube, ID=Inner Diameter, IT=Instrument Tube. Equation 5.4-1 assumes that the instrument tube is filled with water and that there is no instrument inserted. A rodded GT is any GT that contains a rod from either an RCCA, BPRA or APSRA. The inserted rod refers to either an RCCA, BPRA or APSR rod inserted into the GTs of the assembly. In Three Mile Island Unit 1, the inserted rods had no inner clads. Therefore, the inner and outer diameters of the inner clads were zero.

In the follow rod region of the APSR, the F/M ratio was computed with the above equation but with the *Inserted Rod Inner Clad ID* replaced by *APSR Clad ID*.

Equation 5.4-2. Moderator Area in Central Region of Path B Model

$$CRMA = P^2 - \left(\frac{\pi}{4} \right) \left[(GT \text{ OD})^2 - (GT \text{ ID})^2 + (Inserted \text{ Rod Outer Clad OD})^2 - (Inserted \text{ Rod Inner Clad ID})^2 \right]$$

where: CRMA=Central Region Moderator Area.

Equation 5.4-3. Fuel Pellet Cross Sectional Area in Path A Model

$$FA = (Fuel \text{ Pellet OD})^2 \left(\frac{\pi}{4} \right) \text{ where: FA=Fuel Area}$$

Equation 5.4-4. Moderator Cross Sectional Area in Path A Model

$$MA = P^2 - (FR \text{ Clad } OD)^2 \left(\frac{\pi}{4} \right) \text{ where: } MA = \text{Moderator Area}$$

Equation 5.4-5. Relationship Between Fuel-to-Moderator Unit Volume Ratio in the Explicit Assembly Section and the Path B Model

$$\frac{F}{M} \text{ Ratio} = \frac{x(FA)}{CRMA + x(MA)}$$

where: F/M Ratio is from Equation 5.4-1, FA is from Equation 5.4-3, CRMA is from Equation 5.4-2, MA is from Equation 5.4-4, and x refers to the number of assembly fuel pin lattice cells that must be represented in the Path B Model homogenized region to preserve the fuel-to-moderator unit volume ratio.

Equation 5.4-6. Number of Assembly Fuel Pin Lattice Cells Required in the Homogenized Region of the Path B Model

$$x = \frac{\left(\frac{F}{M} \text{ Ratio} \right) (CRMA)}{FA - \left(\frac{F}{M} \text{ Ratio} \right) (MA)}$$

Equation 5.4-7. Path B Model Homogenized Region Outer Radius

$$\text{Homogenized Region Outer Radius} = \sqrt{\frac{x(P)^2}{\pi} + (\text{Homogenized Region Inner Radius})^2}$$

where: x is from Equation 5.4-6 and the Homogenized Region Inner Radius always refers to the outer radius of the Path B model central region which is always the explicit perimeter of an assembly unit cell that has been converted to a radial perimeter by conserving area.

Equation 5.4-8. Inter-Assembly Spacing Moderator Region Outer Radius

$$\text{IASMR Outer Radius} = \left\{ \left(\frac{(x+1)}{\# \text{ Assembly Lattice Unit Cells}} \left[(\text{Assembly Pitch})^2 - (P)^2 (\# \text{ Assembly Lattice Unit Cells}) \right] \left(\frac{1}{\pi} \right) + \right. \right. \\ \left. \left. (\text{Homogenized Region Outer Radius})^2 \right\}^{1/2}$$

where: IASMR=Inter-Assembly Spacing Moderator Region.

When developing the Path B model for an assembly section that has insertion rods in some or all of the guide tubes, the development should begin with an explicit representation of the insertion rod inserted in the guide tube in the central region of the Path B Model. The remaining dimensions of the Path B Model should then be determined by preserving the fuel-to-moderator unit volume ratio in the explicit assembly section. The neutron absorber unit volume or area in the Path B Model must then be adjusted to preserve the fuel-to-absorber unit volume ratio in the explicit assembly section. This adjustment is made by first determining the neutron absorber area that must exist in the Path B model to preserve the fuel-to-absorber ratio. The existing area of the neutron absorber material in the Path B Model is then adjusted by changing the outer radius dimension of the neutron absorber material. The inner radius dimension of the neutron absorber material (if applicable) is always fixed at its explicit value.

Equation 5.4-9. Fuel-to-Neutron Absorber Unit Volume Ratio (Area) in Fuel Assembly Section

$$\frac{F}{Abs} \text{ Ratio} = \frac{(\# FRs)(Fuel Pellet OD)^2 \left(\frac{\pi}{4}\right)}{(\# Insertion Rods) \left(\frac{\pi}{4}\right) [(Abs Pellet OD)^2 - (Abs Pellet ID)^2]}$$

where: F/Abs Ratio=Fuel-to-Neutron Absorber Ratio in the explicit fuel assembly section, Abs=Neutron Absorber Material, Insertion Rods=refers to the rods of either an RCCA or BPRA that are inserted into the guide tubes in the assembly section.

Equation 5.4-10. Relationship Between Fuel-to-Neutron Absorber Unit Volume Ratio in the Explicit Assembly Section and the Path B Model

$$RAA = \frac{x(Fuel Pellet OD)^2 \left(\frac{\pi}{4}\right)}{\frac{F}{Abs} \text{ Ratio}}$$

where: RAA=Required Absorber Area for Path B Model and F/Abs Ratio is from Equation 5.4-9.

Equation 5.4-11. Adjusted Neutron Absorber Area Outer Diameter for Path B Model

$$\text{Adjusted Neutron Absorber Region OD} = \sqrt{\left(\frac{RAA}{(\pi/4)} + (Abs Pellet ID)^2\right)}$$

where: RAA is from Equation 5.4-10.

5.5. Cycle Irradiation History Layouts for the Three Mile Island Unit 1 Depletion Calculations

The RCCA or APSRA insertion history for an assembly was modeled such that the appropriate axial nodes of the fuel assembly were depleted using the appropriate neutron flux and spectrum over the correct exposure duration. The isotopic inventory may be quite different between fuel assemblies with and without an RCCA or APSRA insertion history. These isotopic inventory differences must be accounted for in the CRC depletion calculations to allow for correct prediction of core k_{eff} values in subsequent CRC reactivity calculations.

In SAS2H, the duration of a depletion calculation may be separated into a number of time steps of variable length. Typically, the length of a depletion calculation was the continuous irradiation time required to go from one CRC statepoint or datapoint to another. To follow the RCCA or APSRA insertion histories, detailed intra-cycle variable irradiation time steps were required. This was due to the fact that the rods of the RCCA or APSRA were only present in a given axial node of an assembly for a given period of exposure during a statepoint depletion calculation. A user specified number of cross section library updates were performed during each time step of an irradiation interval. The CRC depletion calculations always used one cross section library update per time step. The boron letdown curve of the reactor cycle may also be followed by specifying, at each irradiation step, a fraction of the soluble boron concentration defined in the base moderator material specification. This boron concentration was applied uniformly over the irradiation time step. The boron concentration fraction at the mid-point of each irradiation time step was specified in the SAS2H depletion calculations of this analysis to appropriately follow boron letdown curves. Considering the cross section update frequency, the boron letdown data, and the absorber rod assembly insertion histories, the following requirements were applied to determining an appropriate reactor cycle irradiation layout for a fuel assembly:

- The duration of each time step was specified such that a maximum of 80 days of irradiation was not exceeded between cross section updates. The SAS2H calculations in this calculation utilized one cross section update per irradiation step. Therefore, the maximum duration of any time step in any reactor cycle irradiation layout of this calculation did not exceed 80 days. The 80 day limit was an arbitrary limit based on engineering judgement. The 80 day irradiation time step limit should assure that the changes in isotopic concentrations of the system (primarily fuel) did not alter the neutron spectrum radically enough to cause a time step of the depletion calculation to be performed without the availability of cross sections which have been properly weighted with an appropriate neutron spectrum and spatial flux.
- Any radical perturbations in the boron letdown curve were followed by defining irradiation time step duration such that the average boron concentration over each time step is representative of the actual boron letdown. Usually, the 80 day time step limit imposed for cross section update frequency is adequate to properly follow a reactor cycle's boron letdown curve.
- The duration of each time step was specified such that the insertion of an RCCA or APSRA in a given assembly axial node could be modeled for the correct exposure time in terms of EFPD. In SAS2H, there is an option to vary the Path B model between irradiation steps as long as the number of radial zones in the Path B models of a given SAS2H calculation remain the same. Therefore, an

assembly axial node represented in a given SAS2H statepoint depletion calculation that has an RCCA or APSRA insertion history for a specified period of exposure, that is less than the total exposure covered by the statepoint depletion calculation, was modeled by changing the Path B model from one representing the insertion of the RCCA or APSRA to one representing the removal of the RCCA or APSRA at the appropriate time step corresponding to the RCCA or APSRA removal time.

The irradiation time step layout for a given statepoint depletion calculation was developed so that breakpoints existed between irradiation time steps that allowed for the appropriate removal or insertion of the RCCA or APSRA to obtain the correct neutron spectrum for each axial node of the assembly. The complexity of the irradiation time step layout for a given statepoint calculation was proportional to the number of axial nodes being modeled and the frequency of RCCA or APSRA movement during the assembly depletion. The time steps developed to model RCCA or APSRA insertion histories were also designed to encompass the cross section update and boron letdown requirements. A software routine entitled "RLAYOUT" was written to automate the development of appropriate irradiation time step layouts for the statepoint depletion calculations of an assembly having an RCCA or APSRA insertion history. The RLAYOUT software routine is described in Attachment III of Reference 7.3.

The RLAYOUT software routine was only utilized to determine the irradiation time step layouts for the CRC depletion calculations that contained an RCCA or APSRA insertion history. A single assembly may have had a combination of CRC calculations that either required or did not require the RLAYOUT developed irradiation time step layouts. For the CRC depletion calculations that did not require the consideration of an RCCA or APSRA insertion history, the irradiation time step layouts were developed by considering the cross section update frequency and the boron letdown data. Tables 5.5-1 through 5.5-4 contain the CRC depletion calculation time step layouts for each Three Mile Island Unit 1 reactor cycle that was relevant to the CRC depletion calculations documented in this calculation file which did not have an RCCA or APSRA insertion history. The mid-step boron concentrations presented in Tables 5.5-1 through 5.5-4 were obtained from linear interpolation (using the Excel spreadsheet) between the measured values presented in Table 5.2.7-1.

The RLAYOUT developed irradiation time step layouts for the assemblies which had an RCCA or APSRA insertion history are presented in Tables 5.5-5 through 5.5-13. The boron letdown data utilized by RLAYOUT in developing these irradiation layouts were obtained from linear interpolation and the data presented in Table 5.2.7-1. The RCCA or APSRA insertion times utilized by RLAYOUT in developing these irradiation layouts were obtained from Tables 5.2.9-1 through 5.2.9-9.

Table 5.5-1. Irradiation Layout for Cycle 2 of Three Mile Island Unit 1

Depletion: BOC to EOC		Time Step Length: 64.05 days	Number of Time Steps: 4
Time Step	Mid-Step EFPD	Mid-Step ppmB	
1	32.025	791.768	
2	96.075	606.304	
3	160.125	420.435	
4	224.175	234.409	

Table 5.5-2. Irradiation Layout for Cycle 3 of Three Mile Island Unit 1

Depletion: BOC to 142.8 EFPD		Time Step Length: 71.4 days	Number of Time Steps: 2
Time Step	Mid-Step EFPD	Mid-Step ppmB	
1	35.700	657.612	
2	107.100	452.836	
Depletion: 142.8 EFPD to EOC		Time Step Length: 72.15 days	Number of Time Steps: 2
Time Step	Mid-Step EFPD	Mid-Step ppmB	
1	178.875	245.445	
2	251.025	42.086	

Table 5.5-3. Irradiation Layout for Cycle 4 of Three Mile Island Unit 1

Depletion: BOC to 126.6 EFPD		Time Step Length: 63.3 days	Number of Time Steps: 2
Time Step	Mid-Step EFPD	Mid-Step ppmB	
1	31.650	746.114	
2	94.950	568.140	
Depletion: 126.6 EFPD to EOC		Time Step Length: 73.7 days	Number of Time Steps: 2
Time Step	Mid-Step EFPD	Mid-Step ppmB	
1	163.450	375.340	
2	237.150	167.980	

Table 5.5-4. Irradiation Layout for Cycle 5 of Three Mile Island Unit 1

Depletion: BOC to 114.4 EFPD		Time Step Length: 57.2 days	Number of Time Steps: 2
Time Step	Mid-Step EFPD	Mid-Step ppmB	
1	28.600	738.632	
2	85.800	572.896	
Depletion: 114.4 EFPD to EOC		Time Step Length: 62.67 days	Number of Time Steps: 3
Time Step	Mid-Step EFPD	Mid-Step ppmB	
1	145.735	398.454	
2	208.405	216.457	
3	271.075	34.461	

Title: CRC Depletion Calculations for Three Mile Island Unit 1
 Document Identifier: B00000000-01717-0210-00007 REV 00

Table 5.5-7. RLAYOUT Developed Irradiation Layout for Assembly B21a of Three Mile Island Unit 1

IRRADIATION LAYOUT FOR ASSEMBLY: B21a
 Cycle-05, .0 EFPD to Cycle-05, 114.4 EFPD Statepoint Calculation

Irradiation Step Number	Step Duration (EFPD)	Exposure at End of Step (EFPD)	Mid-Step Boron Concentration (ppm)
1	3.00	3.00	816.7
2	43.60	46.60	749.6
3	56.70	103.30	604.1
4	11.10	114.40	506.2

NODAL ROD ASSEMBLY INSERTION LAYOUT FOR FUEL ASSEMBLY: B21a

COLUMN A: Cycle-05, .0 EFPD to Cycle-05, 114.4 EFPD Statepoint Calculation

X = Rod assembly inserted in corresponding node during the irradiation step

NODE #	A			
	1	2	3	4
1	X	X	X	
2	X	X		
3	X			
4				
5				
6				
7				
8				
9				
10				
11				
12				
13				
14				
15				
16				
17				
18				

Title: CRC Depletion Calculations for Three Mile Island Unit 1
 Document Identifier: B00000000-01717-0210-00007 REV 00

Table 5.5-11. RLAYOUT Developed Irradiation Layout for Assembly D20a of Three Mile Island Unit 1

IRRADIATION LAYOUT FOR ASSEMBLY: D20a
 Cycle-05, .0 EFPD to Cycle-05, 114.4 EFPD Statepoint Calculation

Irradiation Step Number	Step Duration (EFPD)	Exposure at End of Step (EFPD)	Mid-Step Boron Concentration (ppm)
1	2.90	2.90	816.8
2	43.20	46.10	750.4
3	57.00	103.10	605.2
4	11.30	114.40	506.5

NODAL ROD ASSEMBLY INSERTION LAYOUT FOR FUEL ASSEMBLY: D20a

COLUMN A: Cycle-05, .0 EFPD to Cycle-05, 114.4 EFPD Statepoint Calculation

X = Rod assembly inserted in corresponding node during the irradiation step

NODE #	A			
	1	2	3	4
1	X	X	X	
2	X	X		
3	X			
4				
5				
6				
7				
8				
9				
10				
11				
12				
13				
14				
15				
16				
17				
18				

Table 5.5-13. RLAYOUT Developed Irradiation Layout for Assembly D25a of Three Mile Island Unit 1

IRRADIATION LAYOUT FOR ASSEMBLY: D25a
 Cycle-05, .0 EFPD to Cycle-05, 114.4 EFPD Statepoint Calculation

Irradiation Step Number	Step Duration (EFPD)	Exposure at End of Step (EFPD)	Mid-Step Boron Concentration (ppm)
1	3.00	3.00	816.7
2	43.70	46.70	749.4
3	56.70	103.40	603.9
4	11.00	114.40	506.0

NODAL ROD ASSEMBLY INSERTION LAYOUT FOR FUEL ASSEMBLY: D25a

COLUMN A: Cycle-05, .0 EFPD to Cycle-05, 114.4 EFPD Statepoint Calculation

X = Rod assembly inserted in corresponding node during the irradiation step

NODE #	A			
	1	2	3	4
1	X	X	X	
2	X	X		
3	X			
4				
5				
6				
7				
8				
9				
10				
11				
12				
13				
14				
15				
16				
17				
18				

5.6. The Commercial Reactor Assembly Follow Taskmaster (CRAFT) Software Routine and Usage

The CRAFT software routine directed the performance of the assembly depletion and decay calculations relevant to CRC evaluations. The CRAFT software routine generated input decks for the SAS2H control module of the SCALE modular code system based on user-defined input which described the fuel assembly's specifications and irradiation history. Appropriate isotopic concentrations relevant to both the CRC evaluations containing the fuel assembly and subsequent depletion and decay calculations of the fuel assembly were extracted and stored by CRAFT as it generated and executed the SAS2H cases required to simulate the complete fuel assembly irradiation history.

The CRAFT software routine was developed with a high degree of flexibility to provide for the depletion and decay of fuel assemblies that have widely varying features under flexible core operating conditions. The following listing describes some of the capabilities and usage of CRAFT.

- The CRAFT software routine generates and executes appropriate SAS2H cases required to perform a prescribed depletion and decay sequence for a fuel assembly. The depletion and decay sequence is orchestrated from the BOC statepoint calculation of the initial prescribed insertion cycle through the final statepoint calculation of the last prescribed insertion cycle. The CRAFT software routine extracts and saves fuel and burnable poison isotopics at each statepoint, including BOC statepoints, during the fuel assembly's depletion and decay sequence. A certain number of the generated isotopics in the depleted fuel composition obtained from a SAS2H calculation are not used in the initial charge composition to the next SAS2H calculation due to a lack of cross section data in the specified SAS2H master cross section library. The CRAFT software routine provides a listing of the fuel isotopics from the output of a SAS2H calculation which are not used in the initial charge to the next SAS2H calculation. The isotopics left out of the initial charge are fission products whose reactivity worth is small relative to the isotopics retained in the initial charge composition. The listing of excluded initial charge isotopics allows for a determination of the impact upon the reactivity worth of the initial fuel composition in the subsequent depletion calculation.
- Any assembly design may be analyzed within the bounds of the SAS2H control module through the use of the CRAFT software routine.
- A spacer grid modeling technique is available with the CRAFT software routine. The modeling technique homogenizes the spacer grid material throughout the moderator of the fuel assembly by utilizing a user-defined spacer material and spacer material volume fraction in the moderator. The available spacer grid materials include the following-- ZIRC-4, INCONEL, SS316, SS316S, SS304, SS304S. Any volume fraction of spacer material in the moderator may be specified (including zero).
- The fuel cladding, burnable poison rod cladding, or control rod cladding in the CRAFT calculation may be designated as any of the following materials-- ZIRC-4, INCONEL, SS316, SS316S, SS304, SS304S.

- The insertion of a burnable poison rod assembly (BPRA) during the irradiation of the fuel assembly may be modeled in the CRAFT calculation. Up to 10 unique BPRA designs may be specified for use during the depletion of a fuel assembly. Any type of BPRA design may be specified. The default burnable poison (BP) material for use in CRAFT calculation is $\text{Al}_2\text{O}_3\text{-B}_4\text{C}$. However, any arbitrary BP material may be specified for use in a BPRA design. A maximum of 10 unique BP materials may be specified. A maximum of 20 unique elements or isotopes may be specified in any given BP material. A BPRA may be inserted in any reactor cycle specified in the CRAFT calculation. Only one BPRA design may be specified per cycle. The position of the BPRA in the fuel assembly is specified by identifying the top and bottom axial nodes of the BP material. The BPRA remains fixed during a given reactor cycle. The depletion of the BP material is tracked during the CRAFT calculation. The appropriate depleted BP material is utilized in the statepoint calculations for a given reactor cycle. Depleted BP material isotopic concentrations are also retained for use in subsequent mid-cycle CRC statepoint reactivity calculations.
- The insertion of an RCCA during the irradiation of the fuel assembly may be modeled in the CRAFT calculation. Up to 10 unique RCCA designs may be specified for use during the depletion of a fuel assembly. Any type of RCCA design may be specified. Any arbitrary control rod (CR) absorber material may be specified for use in an RCCA design. A maximum of 10 unique CR absorber materials may be specified. A maximum of 10 unique elements or isotopes may be specified in any given CR absorber material. An RCCA may be inserted in any reactor cycle specified in the CRAFT calculation. Multiple RCCA designs may be specified per cycle. The position of the RCCA in the fuel assembly is specified by identifying the top and bottom axial nodes of sections of the fuel assembly which contain the CR absorber material. The RCCA position may be changed between each irradiation step of a SAS2H calculation generated by CRAFT. The RCCA design may also be changed between any two CRC statepoint depletion calculations in a given reactor cycle.
- A fuel assembly may be inserted in a maximum of 10 reactor cycles during a CRAFT calculation.
- A maximum of 20 statepoints or datapoints (beginning of cycle (BOC)), is always considered a statepoint) may be specified in any given reactor cycle in a CRAFT calculation.
- A maximum of 23 irradiation steps of variable duration may be specified in any given SAS2H depletion calculation that is generated by CRAFT.
- A maximum of 50 axial assembly nodes may be specified for use in a CRAFT calculation. Each axial node may have a unique height.
- The CRAFT software routine utilizes a user-defined input format for fuel temperature, moderator specific volume, and burnup data. The input data must be specified for each axial node in a user-defined nodal format of up to 50 nodes of arbitrary height. The total assembly active fuel height for the input data descriptions may be different than that specified for use in the CRAFT generated SAS2H depletion calculations. Depending on the users needs, the fuel temperature, moderator specific volume, and burnup input data may be specified in a different nodal format each time a set of this input data is provided. Nominal fuel temperature input data representing full-power reactor

operation must be provided in units of degrees Fahrenheit for each node in each CRC statepoint depletion calculation that will be generated by CRAFT. Nominal moderator specific volume input data representing full-power reactor operation must be provided in units of cubic feet per pound for each node in each statepoint calculation that will be generated by CRAFT. The nodal average burnup input data must be provided in units of GWd/MTU for each node at each statepoint or datapoint including all BOC statepoints. All burnup input data that is specified must be cumulative from the initial insertion of the fuel assembly in the reactor.

- A continuation CRAFT calculation for an assembly may be initiated from any statepoint in any reactor cycle if all of the nodal consolidated output files ("*.cut" files) from the statepoint calculation immediately preceding the continuation calculation exist in the CRAFT execution directory.

Additional information on the CRAFT software routine is provided in the CRAFT user information in Attachment I of Reference 7.6. Instructions on how to develop CRAFT input decks and execute CRAFT calculations are also provided there. This attachment also discusses specific modeling procedures and details relevant to the SAS2H fuel assembly depletion calculations which were generated by CRAFT. Version 4 of CRAFT was used to run all cases except assembly C21, which was run with Version 5.

5.7. Input and Output Filename Descriptions for CRAFT and SAS2H

The CRAFT code generated five types of files identified as either "*.input", "*.output", "*.cut", "*.msgs", or "*.notes", where the "*" is the base file set identifier for the statepoint depletion calculation of interest. The "*.cut" and "*.notes" files were the only files that had to be retained for CRC reactivity evaluations and documentation purposes. All files were generated in the working directory in which the CRAFT calculation was performed.

All CRAFT generated filenames utilized the following format-- "{Base File Set Identifier}.{suffix}", where the suffix corresponded to one of the five file types previously mentioned, and the base file set identifier was a 25 character name containing essential information necessary to uniquely identify each CRAFT generated SAS2H depletion calculation.

The base file set identifier for each statepoint depletion calculation contained the following information:

1. reactor identifier (three character)
2. one-eighth core symmetry assembly number in current reactor cycle (two digit)
3. axial node number (node 1 is always the top node) (two digit)
4. reactor cycle number in which the SAS2H calculation starts (two character)
5. EFPD statepoint at which the SAS2H calculation starts (three digit)
6. reactor cycle number in which the SAS2H calculation ends (two character)
7. EFPD statepoint at which the SAS2H calculation ends (three digit).

The format of the base file set identifier was as follows where the numbers identified as #{number} correspond to one of the seven items previously listed-- #1 A #2 N #3 DC #4 T #5 AC #6 T #7. The

letters contained in the base file set identifier were presented explicitly as shown in the previous format. The base file set identifier did not contain any spaces.

The "*.input" files each contained a CRAFT generated SAS2H input deck. The "*.output" files each contained a complete SAS2H depletion calculation output file. The "*.cut" files each contained the corresponding SAS2H input deck followed by an output extraction from the final ORIGEN-S pass of the SAS2H depletion calculation, which contained data relevant to subsequent CRC reactivity calculations. The "*.msgs" files each contained the standard run-time messages associated with the SAS2H calculations. The "*.notes" files each contained a listing of the isotopes and associated concentrations which were left behind in generating the initial charge fuel composition for the next continuation SAS2H calculation. The "*.notes" files were only created for CRAFT generated SAS2H calculations which were continuation depletion calculations. The "*.cut" and "*.notes" files contained all of the information required to perform CRC reactivity evaluations or repeat calculations as necessary for quality assurance purposes. The remainder of the CRAFT generated files were discarded once the "*.cut" and "*.notes" files were generated and retained.

6. Results

Depletion calculations for 29 fuel assemblies from Three Mile Island Unit 1 were documented in this calculation. The depleted fuel isotopics for these fuel assemblies had to be calculated at two statepoints in cycle 5 for use in subsequent CRC reactivity calculations. Table 5.2.6-2 identifies the CRC statepoint EFPD values in cycle 5 for which isotopic compositions were required. Table 5.2.6-2 also identifies a number of datapoints at which the depletion calculations were interrupted to update input parameters. Even though the depleted isotopics available at each of the datapoints were not required for subsequent reactivity calculations, they were retained in this calculation file for completeness.

The CRAFT input decks for each assembly depletion were developed in accordance with the instructions presented in Sections 5 and 7 of Attachment I of Reference 7.6. The SAS2H modeling features incorporated in the depletion calculations are described in Attachment I of Reference 7.6. The CRAFT input decks for the assembly depletions documented in this calculation file are provided in Attachment I, as documented in Section 8.

Attachment II (this attachment has been moved to Reference 7.7) contains the CRAFT generated consolidated SAS2H output files for the depletion calculations documented in this analysis as identified in the attachment listing of Section 8. The consolidated output files contain the following information:

- time/date stamp for when the SAS2H depletion calculation was performed
- echo of the SAS2H input deck generated by CRAFT
- the output extraction of information pertinent to CRC evaluations from the final ORIGEN-S calculation of the SAS2H depletion calculation.

Between CRC statepoints or datapoints in the depletion sequence for a fuel assembly axial region, a new SAS2H input deck had to be created using the fuel isotopic results from the previous calculation as the initial charge. Since the 44-group master cross section library utilized in the SAS2H depletion calculations of this analysis had a reduced isotopic inventory relative to the ORIGEN-S cross section library, a number of isotopes present in the ORIGEN-S output could not be transferred to the initial fuel charge of the subsequent SAS2H depletion calculation. The isotopic inventory in the ORIGEN-S output which could not be propagated to the continuation SAS2H depletion calculation did not significantly affect the integral reactivity or the energy dependent neutron spectrum as documented in Section 4.9.1 of Attachment I of Reference 7.6. The non-propagated isotopic inventory was written to a file entitled "{depletion case identifier}.notes" to allow for subsequent analysis of the impact of excluding these isotopes in the initial charge to the continuation SAS2H depletion calculation. The "*.notes" files are contained in Attachment III as documented in Section 8.

Isotopic results for the set of 29 principal isotopes identified in Table 6-1 were tabulated for each axial node of each fuel assembly at each CRC statepoint other than beginning of life (BOC of first reactor cycle in which the assembly is inserted) statepoint. The program entitled "CRC_DATA_TABULIZER.exe" described in Attachment VI of Reference 7.6, was used to create the principal isotope result tables documented in this calculation file. Attachment IV (this attachment has been moved to Reference 7.7) contains the principal isotope tabulations for the assemblies documented in this calculation file.

Table 6-1. The Set of 29 Principal Isotopes

Mo-95	Tc-99	Ru-101	Rh-103	Ag-109
Nd-143	Nd-145	Sm-147	Sm-149	Sm-150
Sm-151	Sm-152	Eu-151	Eu-153	Gd-155
U-233	U-234	U-235	U-236	U-238
Np-237	Pu-238	Pu-239	Pu-240	Pu-241
Pu-242	Am-241	Am-242m	Am-243	---

7. References

- 7.1 *SCALE, Version 4.3: Modular Code System for Performing Standardized Computer Analyses for Licensing Evaluation.* User's Manual Volumes 0 through 3, Oak Ridge National Laboratory. Distributed by the Radiation Shielding Information Center, Oak Ridge National Laboratory, Document Number: CCC-545.
- 7.2 *Software Qualification Report for the SCALE Modular Code System Version 4.3.* SCALE Version 4.3 Computer Software Configuration Item (CSCD): 30011 V4.3. Document Identifier Number (DI#): 30011-2002 REV 00, Civilian Radioactive Waste Management System (CRWMS) Management and Operating Contractor (M&O).
- 7.3 *CRC Depletion Calculations for the Rodded Assemblies in Batches 1, 2, 3, and IX of Crystal River Unit 3.* DI#: BBA000000-01717-0200-00040 REV 00, CRWMS M&O.

- 7.4 *MCNP – A General Monte Carlo N-Particle Transport Code, Version 4B.* User's Manual, Los Alamos National Laboratory. Document Number: LA-12625-M. Issued March 1, 1997.
- 7.5 *Summary Report of Commercial Reactor Criticality Data for Three Mile Island Unit 1.* DI#: B00000000-01717-5705-00069 REV 00, CRWMS M&O.
- 7.6 *CRC Depletion Calculations for McGuire Unit 1.* DI#: B00000000-01717-0210-00003 REV 00, CRWMS M&O.
- 7.7 *CRC Depletion Calculations for Three Mile Island Unit 1 (DI#: B00000000-01717-0210-00007 REV 00) - Attachments I through IV - 1 Data Cartridge.* Batch Number: MOY-980428-17.

8. Attachments

Attachments I through IV referenced throughout this design analysis are listed in Table 8-1. These attachments have been moved to Reference 7.7. Attachment I contains the CRAFT input decks for the assembly depletions. Attachment II contains the "*.cut" files for the assembly depletion calculations. Attachment III contains the "*.notes" files for the assembly depletion calculations. These attachments were written in ASCII format to an attachment tape (Ref. 7.7). Detailed listings of the content of the Attachments on the tape are provided in their corresponding hard copy attachment location in this calculation file. The listing of the content of the Attachments contain the following information, as appropriate, for each of the files that were written to the tape:

- the directory and filename as taken from the HP workstation
- the corresponding filename on the attachment tape
- the date that the file was created on the HP workstation or personal computer
- the size of the file on the HP workstation or personal computer.

The tape containing Attachments I to IV was written using the HP Colorado Model T1000e External Parallel Port Backup System for personal computers.

Table 8-1. Attachment Listing

Attachment #	# of Pages	Creation Date	Description
I	1 (Listing of tape content)	3/18/98	CRAFT input decks for depletion calculations (moved to Reference 7.7)
II	29 (Listing of tape content)	4/24/97	"*.cut" consolidated output files for the depletion calculations (moved to Reference 7.7)
III	21 (Listing of tape content)	4/24/97	"*.notes" files for the depletion calculations (moved to Reference 7.7)
IV	1 (Listing of tape content)	3/18/98	principal isotope tabulized results for the depletion calculations (moved to Reference 7.7)

Title: CRC Depletion Calculations for TMI Unit 1

Document Identifier: B00000000-01717-0210-00007 REV 00

Attachment I, Page 1 of 1

This attachment contains the CRAFT input decks for the depletion calculations for Three Mile Island Unit 1. The input decks are contained on an attachment tape of this calculation file (the attachment tape has been moved to Reference 7.7). The information contained in this hard-copy representation of Attachment I is a listing of the various CRAFT input deck files and their attributes. The file sizes listed in the following table are the file sizes as they appear on the Hewlett Packard (HP) Series 9000 workstation. The HP file sizes differ from the file sizes on the attachment tape due to the difference in the block sizes between the HP and the personal computer. The tape containing Attachment I was written using the Colorado Model T1000e External Parallel Port Backup System for personal computers.

Filename	File Type	File Size (Bytes)	Date File Copied to Tape
B15i.dat	ASCII	20,490	03/18/98
B21ai.dat	ASCII	18,369	03/18/98
B21i.dat	ASCII	19,289	03/18/98
B28i.dat	ASCII	16,535	03/18/98
B29i.dat	ASCII	19,265	03/18/98
C08i.dat	ASCII	14,054	03/18/98
C15ai.dat	ASCII	14,110	03/18/98
C15i.dat	ASCII	14,081	03/18/98
C20i.dat	ASCII	14,105	03/18/98
C21i.dat	ASCII	16,987	03/18/98
C25i.dat	ASCII	14,085	03/18/98
C28i.dat	ASCII	14,074	03/18/98
C29i.dat	ASCII	16,711	03/18/98
D08i.dat	ASCII	9,629	03/18/98
D15i.dat	ASCII	9,622	03/18/98
D20ai.dat	ASCII	11,537	03/18/98
D20i.dat	ASCII	9,472	03/18/98
D21i.dat	ASCII	12,147	03/18/98
D25ai.dat	ASCII	11,515	03/18/98
D25i.dat	ASCII	9,460	03/18/98
D27i.dat	ASCII	9,637	03/18/98
D28i.dat	ASCII	9,627	03/18/98
E08i.dat	ASCII	5,328	03/18/98
E15i.dat	ASCII	5,325	03/18/98
E20i.dat	ASCII	5,338	03/18/98
E21i.dat	ASCII	5,324	03/18/98
E25i.dat	ASCII	5,327	03/18/98
E27i.dat	ASCII	5,330	03/18/98
E28i.dat	ASCII	5,322	03/18/98

Title: CRC Depletion Calculations for TMI Unit 1

Document Identifier: B00000000-01717-0210-00007 REV 00

Attachment II, Page 1 of 29

This attachment contains the consolidated SAS2H output files that were generated by CRAFT during the depletion calculations for Three Mile Island Unit 1. These files are referred to as "*.cut" files due to their ".cut" extension. The "*.cut" files are contained on an attachment tape of this calculation file (the attachment tape has been moved to Reference 7.7). The information contained in this hard-copy representation of Attachment II is a listing of the various "*.cut" files and their attributes. The file sizes listed in the following table are the file sizes as they appear on the Hewlett Packard (HP) Series 9000 workstation. The HP file sizes differ from the file sizes on the attachment tape due to the difference in the block sizes between the HP and the personal computer. The tape containing Attachment II was written using the Colorado Model T1000e External Parallel Port Backup System for personal computers.

Computer File Name	Tape Backup File Name	File Date (Output)	File Size (Bytes)	File Type (Format)
B15/TMIA12N01DC04T000AC04T126.cut	aI.If1	Feb 24 1997	292819	ASCII
B15/TMIA12N01DC04T126AC05T000.cut	aI.If2	Feb 24 1997	124114	ASCII
B15/TMIA12N02DC04T000AC04T126.cut	aI.If3	Feb 24 1997	296811	ASCII
B15/TMIA12N02DC04T126AC05T000.cut	aI.If4	Feb 24 1997	126134	ASCII
B15/TMIA12N03DC04T000AC04T126.cut	aI.If5	Feb 24 1997	298886	ASCII
B15/TMIA12N03DC04T126AC05T000.cut	aI.If6	Feb 24 1997	126553	ASCII
B15/TMIA12N04DC04T000AC04T126.cut	aI.If7	Feb 24 1997	299550	ASCII
B15/TMIA12N04DC04T126AC05T000.cut	aI.If8	Feb 24 1997	126553	ASCII
B15/TMIA12N05DC04T000AC04T126.cut	aI.If9	Feb 24 1997	299965	ASCII
B15/TMIA12N05DC04T126AC05T000.cut	aII.f10	Feb 24 1997	126636	ASCII
B15/TMIA12N06DC04T000AC04T126.cut	aII.f11	Feb 24 1997	299799	ASCII
B15/TMIA12N06DC04T126AC05T000.cut	aII.f12	Feb 24 1997	126636	ASCII
B15/TMIA12N07DC04T000AC04T126.cut	aII.f13	Feb 24 1997	299965	ASCII
B15/TMIA12N07DC04T126AC05T000.cut	aII.f14	Feb 24 1997	126636	ASCII
B15/TMIA12N08DC04T000AC04T126.cut	aII.f15	Feb 24 1997	299467	ASCII
B15/TMIA12N08DC04T126AC05T000.cut	aII.f16	Feb 24 1997	126553	ASCII
B15/TMIA12N09DC04T000AC04T126.cut	aII.f17	Feb 24 1997	298969	ASCII
B15/TMIA12N09DC04T126AC05T000.cut	aII.f18	Feb 24 1997	126553	ASCII
B15/TMIA12N10DC04T000AC04T126.cut	aII.f19	Feb 24 1997	297641	ASCII
B15/TMIA12N10DC04T126AC05T000.cut	aII.f20	Feb 24 1997	126300	ASCII
B15/TMIA12N11DC04T000AC04T126.cut	aII.f21	Feb 24 1997	297305	ASCII
B15/TMIA12N11DC04T126AC05T000.cut	aII.f22	Feb 24 1997	126217	ASCII
B15/TMIA12N12DC04T000AC04T126.cut	aII.f23	Feb 24 1997	297291	ASCII
B15/TMIA12N12DC04T126AC05T000.cut	aII.f24	Feb 24 1997	126217	ASCII
B15/TMIA12N13DC04T000AC04T126.cut	aII.f25	Feb 24 1997	297627	ASCII
B15/TMIA12N13DC04T126AC05T000.cut	aII.f26	Feb 24 1997	126383	ASCII
B15/TMIA12N14DC04T000AC04T126.cut	aII.f27	Feb 24 1997	299052	ASCII
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B15/TMIA12N16DC04T126AC05T000.cut	aII.f32	Feb 24 1997	126553	ASCII
B15/TMIA12N17DC04T000AC04T126.cut	aII.f33	Feb 24 1997	296645	ASCII
B15/TMIA12N17DC04T126AC05T000.cut	aII.f34	Feb 24 1997	126134	ASCII
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B15/TMIA12N18DC04T126AC05T000.cut	aII.f36	Feb 24 1997	124031	ASCII
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B15/TMIA14N08DC05T000AC05T114.cut	aII.f44	Feb 24 1997	148699	ASCII
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B15/TMIA14N10DC05T000AC05T114.cut	aII.f46	Feb 24 1997	148201	ASCII
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B15/TMIA14N12DC05T000AC05T114.cut	aII.f48	Feb 24 1997	148201	ASCII
B15/TMIA14N13DC05T000AC05T114.cut	aII.f49	Feb 24 1997	148284	ASCII
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B15/TMIA14N16DC05T000AC05T114.cut	aII.f52	Feb 24 1997	148616	ASCII
B15/TMIA14N17DC05T000AC05T114.cut	aII.f53	Feb 24 1997	147371	ASCII

Title: CRC Depletion Calculations for TMI Unit 1

Document Identifier: B00000000-01717-0210-00007 REV 00

Attachment II, Page 2 of 29

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B15/TMIA18N06DC03T000AC03T142.cut	aII.f83	Feb 24 1997	320765	ASCII
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B15/TMIA18N07DC03T142AC04T000.cut	aII.f86	Feb 24 1997	160515	ASCII
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B15/TMIA18N11DC03T142AC04T000.cut	aII.f94	Feb 24 1997	163398	ASCII
B15/TMIA18N12DC03T000AC03T142.cut	aII.f95	Feb 24 1997	324237	ASCII
B15/TMIA18N12DC03T142AC04T000.cut	aII.f96	Feb 24 1997	163398	ASCII
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B15/TMIA18N13DC03T142AC04T000.cut	aII.f98	Feb 24 1997	162976	ASCII
B15/TMIA18N14DC03T000AC03T142.cut	aII.f99	Feb 24 1997	325233	ASCII
B15/TMIA18N14DC03T142AC04T000.cut	aII.f.100	Feb 24 1997	162976	ASCII
B15/TMIA18N15DC03T000AC03T142.cut	aII.f.101	Feb 24 1997	317402	ASCII
B15/TMIA18N15DC03T142AC04T000.cut	aII.f.102	Feb 24 1997	158127	ASCII
B15/TMIA18N16DC03T000AC03T142.cut	aII.f.103	Feb 24 1997	317402	ASCII
B15/TMIA18N16DC03T142AC04T000.cut	aII.f.104	Feb 24 1997	158127	ASCII
B15/TMIA18N17DC03T000AC03T142.cut	aII.f.105	Feb 24 1997	316084	ASCII
B15/TMIA18N17DC03T142AC04T000.cut	aII.f.106	Feb 24 1997	157642	ASCII
B15/TMIA18N18DC03T000AC03T142.cut	aII.f.107	Feb 24 1997	311768	ASCII
B15/TMIA18N18DC03T142AC04T000.cut	aII.f.108	Feb 24 1997	155563	ASCII

Computer File Name	Tape Backup File Name	File Date (Output)	File Size (Bytes)	File Type (Format)
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B21/TMIA01N02DC05T000AC05T114.cut	aII.f.110	Feb 24 1997	147699	ASCII
B21/TMIA01N03DC05T000AC05T114.cut	aII.f.111	Feb 24 1997	148533	ASCII
B21/TMIA01N04DC05T000AC05T114.cut	aII.f.112	Feb 24 1997	148699	ASCII
B21/TMIA01N05DC05T000AC05T114.cut	aII.f.113	Feb 24 1997	148699	ASCII
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Title: CRC Depletion Calculations for TMI Unit 1

Document Identifier: B00000000-01717-0210-00007 REV 00

Attachment II, Page 3 of 29

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Waste Package Operations

Engineering Calculation Attachment

Title: CRC Depletion Calculations for TMI Unit 1

Document Identifier: B00000000-01717-0210-00007 REV 00

Attachment II, Page 4 of 29

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Waste Package Operations

Engineering Calculation Attachment

Title: CRC Depletion Calculations for TMI Unit 1

Document Identifier: B00000000-01717-0210-00007 REV 00

Attachment II, Page 5 of 29

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Title: CRC Depletion Calculations for TMI Unit 1

Document Identifier: B0000000-01717-0210-00007 REV 00

Attachment II, Page 7 of 29

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Title: CRC Depletion Calculations for TMI Unit 1

Document Identifier: B00000000-01717-0210-00007 REV 00

Attachment II, Page 8 of 29

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Waste Package Operations

Engineering Calculation Attachment

Title: CRC Depletion Calculations for TMI Unit 1

Document Identifier: B00000000-01717-0210-00007 REV 00

Attachment II, Page 9 of 29

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Title: CRC Depletion Calculations for TMI Unit 1

Document Identifier: B00000000-01717-0210-00007 REV 00

Attachment II, Page 10 of 29

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Title: CRC Depletion Calculations for TMI Unit 1

Document Identifier: B00000000-01717-0210-00007 REV 00

Attachment II, Page 11 of 29

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Title: CRC Depletion Calculations for TMI Unit 1

Document Identifier: B0000000-01717-0210-00007 REV 00

Attachment II, Page 12 of 29

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Title: CRC Depletion Calculations for TMI Unit 1

Document Identifier: B00000000-01717-0210-00007 REV 00

Attachment II, Page 14 of 29

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Title: CRC Depletion Calculations for TMI Unit 1

Document Identifier: B0000000-01717-0210-00007 REV 00

Attachment II, Page 15 of 29

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Title: CRC Depletion Calculations for TMI Unit 1

Document Identifier: B00000000-01717-0210-00007 REV 00

Attachment II, Page 16 of 29

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Computer

Tape Backup

File Date

File Size

File Type

Title: CRC Depletion Calculations for TMI Unit 1

Document Identifier: B0000000-01717-0210-00007 REV 00

Attachment II, Page 17 of 29

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Title: CRC Depletion Calculations for TMI Unit 1

Document Identifier: B00000000-01717-0210-00007 REV 00

Attachment II, Page 18 of 29

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Title: CRC Depletion Calculations for TMI Unit 1

Document Identifier: B00000000-01717-0210-00007 REV 00

Attachment II, Page 19 of 29

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Title: CRC Depletion Calculations for TMI Unit 1

Document Identifier: B00000000-01717-0210-00007 REV 00

Attachment II, Page 20 of 29

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Title: CRC Depletion Calculations for TMI Unit 1

Document Identifier: B0000000-01717-0210-00007 REV 00

Attachment II, Page 21 of 29

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Title: CRC Depletion Calculations for TMI Unit 1

Document Identifier: B00000000-01717-0210-00007 REV 00

Attachment II, Page 22 of 29

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Title: CRC Depletion Calculations for TMI Unit 1

Document Identifier: B0000000-01717-0210-00007 REV 00

Attachment II, Page 23 of 29

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Title: CRC Depletion Calculations for TMI Unit 1

Document Identifier: B00000000-01717-0210-00007 REV 00

Attachment II, Page 24 of 29

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Title: CRC Depletion Calculations for TMI Unit 1

Document Identifier: B00000000-01717-0210-00007 REV 00

Attachment II, Page 25 of 29

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D27/TMIA27N09DC04T000AC04T126.cut	aIIf1.673	Feb 25 1997	286506	ASCII
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Title: CRC Depletion Calculations for TMI Unit 1

Document Identifier: B0000000-01717-0210-00007 REV 00

Attachment II, Page 26 of 29

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D28/TMIA11N04DC05T000AC05T114.cut	aIIf1.696	Feb 25 1997	144059	ASCII
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D28/TMIA11N07DC05T000AC05T114.cut	aIIf1.699	Feb 25 1997	144142	ASCII
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D28/TMIA11N13DC05T000AC05T114.cut	aIIf1.705	Feb 25 1997	144059	ASCII
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D28/TMIA28N17DC04T126AC05T000.cut	aIIf1.744	Feb 25 1997	121110	ASCII
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Title: CRC Depletion Calculations for TMI Unit 1

Document Identifier: B00000000-01717-0210-00007 REV 00

Attachment II, Page 27 of 29

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E08/TMIA08N12DC05T000AC05T114.cut	aIIf1.758	Feb 25 1997	138297	ASCII
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E08/TMIA08N16DC05T000AC05T114.cut	aIIf1.762	Feb 25 1997	137633	ASCII
E08/TMIA08N17DC05T000AC05T114.cut	aIIf1.763	Feb 25 1997	136886	ASCII
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Computer File Name	Tape Backup File Name	File Date (Output)	File Size (Bytes)	File Type (Format)
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Computer File Name	Tape Backup File Name	File Date (Output)	File Size (Bytes)	File Type (Format)
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Waste Package Operations

Engineering Calculation Attachment

Title: CRC Depletion Calculations for TMI Unit 1

Document Identifier: B00000000-01717-0210-00007 REV 00

Attachment II, Page 28 of 29

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Computer File Name	Tape Backup File Name	File Date (Output)	File Size (Bytes)	File Type (Format)
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E25/TMIA25N07DC05T000AC05T114.cut	aIIf1.825	Feb 24 1997	138795	ASCII
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Computer File Name	Tape Backup File Name	File Date (Output)	File Size (Bytes)	File Type (Format)
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E27/TMIA27N03DC05T000AC05T114.cut	aIIf1.839	Feb 24 1997	138795	ASCII
E27/TMIA27N04DC05T000AC05T114.cut	aIIf1.840	Feb 24 1997	139044	ASCII
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E27/TMIA27N06DC05T000AC05T114.cut	aIIf1.842	Feb 24 1997	139193	ASCII
E27/TMIA27N07DC05T000AC05T114.cut	aIIf1.843	Feb 24 1997	139027	ASCII
E27/TMIA27N08DC05T000AC05T114.cut	aIIf1.844	Feb 24 1997	138944	ASCII
E27/TMIA27N09DC05T000AC05T114.cut	aIIf1.845	Feb 24 1997	138944	ASCII
E27/TMIA27N10DC05T000AC05T114.cut	aIIf1.846	Feb 24 1997	138944	ASCII
E27/TMIA27N11DC05T000AC05T114.cut	aIIf1.847	Feb 24 1997	138944	ASCII
E27/TMIA27N12DC05T000AC05T114.cut	aIIf1.848	Feb 24 1997	138944	ASCII
E27/TMIA27N13DC05T000AC05T114.cut	aIIf1.849	Feb 24 1997	139193	ASCII
E27/TMIA27N14DC05T000AC05T114.cut	aIIf1.850	Feb 24 1997	139027	ASCII
E27/TMIA27N15DC05T000AC05T114.cut	aIIf1.851	Feb 24 1997	138944	ASCII
E27/TMIA27N16DC05T000AC05T114.cut	aIIf1.852	Feb 24 1997	138795	ASCII
E27/TMIA27N17DC05T000AC05T114.cut	aIIf1.853	Feb 24 1997	138297	ASCII
E27/TMIA27N18DC05T000AC05T114.cut	aIIf1.854	Feb 24 1997	136720	ASCII

Computer File Name	Tape Backup File Name	File Date (Output)	File Size (Bytes)	File Type (Format)
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E28/TMIA28N02DC05T000AC05T114.cut	aIIf1.856	Feb 24 1997	136388	ASCII
E28/TMIA28N03DC05T000AC05T114.cut	aIIf1.857	Feb 24 1997	137218	ASCII

Title: CRC Depletion Calculations for TMI Unit 1

Document Identifier: B00000000-01717-0210-00007 REV 00

Attachment II, Page 29 of 29

E28/TMIA28N04DC05T000AC05T114.cut	aIIf1.858	Feb 24 1997	137633	ASCII
E28/TMIA28N05DC05T000AC05T114.cut	aIIf1.859	Feb 24 1997	137716	ASCII
E28/TMIA28N06DC05T000AC05T114.cut	aIIf1.860	Feb 24 1997	137716	ASCII
E28/TMIA28N07DC05T000AC05T114.cut	aIIf1.861	Feb 24 1997	137716	ASCII
E28/TMIA28N08DC05T000AC05T114.cut	aIIf1.862	Feb 24 1997	137716	ASCII
E28/TMIA28N09DC05T000AC05T114.cut	aIIf1.863	Feb 24 1997	137716	ASCII
E28/TMIA28N10DC05T000AC05T114.cut	aIIf1.864	Feb 24 1997	137716	ASCII
E28/TMIA28N11DC05T000AC05T114.cut	aIIf1.865	Feb 24 1997	137716	ASCII
E28/TMIA28N12DC05T000AC05T114.cut	aIIf1.866	Feb 24 1997	137716	ASCII
E28/TMIA28N13DC05T000AC05T114.cut	aIIf1.867	Feb 24 1997	137716	ASCII
E28/TMIA28N14DC05T000AC05T114.cut	aIIf1.868	Feb 24 1997	137716	ASCII
E28/TMIA28N15DC05T000AC05T114.cut	aIIf1.869	Feb 24 1997	137633	ASCII
E28/TMIA28N16DC05T000AC05T114.cut	aIIf1.870	Feb 24 1997	137467	ASCII
E28/TMIA28N17DC05T000AC05T114.cut	aIIf1.871	Feb 24 1997	136803	ASCII
E28/TMIA28N18DC05T000AC05T114.cut	aIIf1.872	Feb 24 1997	134479	ASCII

Title: CRC Depletion Calculations for TMI Unit 1

Document Identifier: B00000000-01717-0210-00007 REV 00

Attachment III, Page 1 of 21

This attachment contains the "*.notes" files that were generated by CRAFT during the depletion calculations for Three Mile Island Unit 1. These files are referred to as "*.notes" files due to their ".notes" extension. The "*.notes" files are contained on an attachment tape of this calculation file (the attachment tape has been moved to Reference 7.7). The information contained in this hard-copy representation of Attachment III is a listing of the various "*.notes" files and their attributes. The file sizes listed in the following table are the file sizes as they appear on the Hewlett Packard (HP) Series 9000 workstation. The HP file sizes differ from the file sizes on the attachment tape due to the difference in the block sizes between the HP and the personal computer. The tape containing Attachment II was written using the Colorado Model T1000e External Parallel Port Backup System for personal computers.

Computer File Name	Tape Backup File Name	File Date (Output)	File Size (Bytes)	File Type (Format)
B15/TMIA12N01DC04T000AC04T126.notes	aII.If1	Feb 24 1997	9689	ASCII
B15/TMIA12N01DC04T126AC05T000.notes	aII.If2	Feb 24 1997	18706	ASCII
B15/TMIA12N02DC04T000AC04T126.notes	aII.If3	Feb 24 1997	9847	ASCII
B15/TMIA12N02DC04T126AC05T000.notes	aII.If4	Feb 24 1997	19619	ASCII
B15/TMIA12N03DC04T000AC04T126.notes	aII.If5	Feb 24 1997	9956	ASCII
B15/TMIA12N03DC04T126AC05T000.notes	aII.If6	Feb 24 1997	20180	ASCII
B15/TMIA12N04DC04T000AC04T126.notes	aII.If7	Feb 24 1997	9928	ASCII
B15/TMIA12N04DC04T126AC05T000.notes	aII.If8	Feb 24 1997	20332	ASCII
B15/TMIA12N05DC04T000AC04T126.notes	aII.If9	Feb 24 1997	9924	ASCII
B15/TMIA12N05DC04T126AC05T000.notes	aIII.f10	Feb 24 1997	20418	ASCII
B15/TMIA12N06DC04T000AC04T126.notes	aIII.f11	Feb 24 1997	9936	ASCII
B15/TMIA12N06DC04T126AC05T000.notes	aIII.f12	Feb 24 1997	20270	ASCII
B15/TMIA12N07DC04T000AC04T126.notes	aIII.f13	Feb 24 1997	9968	ASCII
B15/TMIA12N07DC04T126AC05T000.notes	aIII.f14	Feb 24 1997	20356	ASCII
B15/TMIA12N08DC04T000AC04T126.notes	aIII.f15	Feb 24 1997	10024	ASCII
B15/TMIA12N08DC04T126AC05T000.notes	aIII.f16	Feb 24 1997	20354	ASCII
B15/TMIA12N09DC04T000AC04T126.notes	aIII.f17	Feb 24 1997	9879	ASCII
B15/TMIA12N09DC04T126AC05T000.notes	aIII.f18	Feb 24 1997	20242	ASCII
B15/TMIA12N10DC04T000AC04T126.notes	aIII.f19	Feb 24 1997	9833	ASCII
B15/TMIA12N10DC04T126AC05T000.notes	aIII.f20	Feb 24 1997	20088	ASCII
B15/TMIA12N11DC04T000AC04T126.notes	aIII.f21	Feb 24 1997	9826	ASCII
B15/TMIA12N11DC04T126AC05T000.notes	aIII.f22	Feb 24 1997	20082	ASCII
B15/TMIA12N12DC04T000AC04T126.notes	aIII.f23	Feb 24 1997	9854	ASCII
B15/TMIA12N12DC04T126AC05T000.notes	aIII.f24	Feb 24 1997	20048	ASCII
B15/TMIA12N13DC04T000AC04T126.notes	aIII.f25	Feb 24 1997	9851	ASCII
B15/TMIA12N13DC04T126AC05T000.notes	aIII.f26	Feb 24 1997	20140	ASCII
B15/TMIA12N14DC04T000AC04T126.notes	aIII.f27	Feb 24 1997	9849	ASCII
B15/TMIA12N14DC04T126AC05T000.notes	aIII.f28	Feb 24 1997	20287	ASCII
B15/TMIA12N15DC04T000AC04T126.notes	aIII.f29	Feb 24 1997	9916	ASCII
B15/TMIA12N15DC04T126AC05T000.notes	aIII.f30	Feb 24 1997	20330	ASCII
B15/TMIA12N16DC04T000AC04T126.notes	aIII.f31	Feb 24 1997	9920	ASCII
B15/TMIA12N16DC04T126AC05T000.notes	aIII.f32	Feb 24 1997	20126	ASCII
B15/TMIA12N17DC04T000AC04T126.notes	aIII.f33	Feb 24 1997	9793	ASCII
B15/TMIA12N17DC04T126AC05T000.notes	aIII.f34	Feb 24 1997	19591	ASCII
B15/TMIA12N18DC04T000AC04T126.notes	aIII.f35	Feb 24 1997	9686	ASCII
B15/TMIA12N18DC04T126AC05T000.notes	aIII.f36	Feb 24 1997	18637	ASCII
B15/TMIA14N01DC05T000AC05T114.notes	aIII.f37	Feb 24 1997	7750	ASCII
B15/TMIA14N02DC05T000AC05T114.notes	aIII.f38	Feb 24 1997	8157	ASCII
B15/TMIA14N03DC05T000AC05T114.notes	aIII.f39	Feb 24 1997	8196	ASCII
B15/TMIA14N04DC05T000AC05T114.notes	aIII.f40	Feb 24 1997	8206	ASCII
B15/TMIA14N05DC05T000AC05T114.notes	aIII.f41	Feb 24 1997	8221	ASCII
B15/TMIA14N06DC05T000AC05T114.notes	aIII.f42	Feb 24 1997	8295	ASCII
B15/TMIA14N07DC05T000AC05T114.notes	aIII.f43	Feb 24 1997	8257	ASCII
B15/TMIA14N08DC05T000AC05T114.notes	aIII.f44	Feb 24 1997	8192	ASCII
B15/TMIA14N09DC05T000AC05T114.notes	aIII.f45	Feb 24 1997	8184	ASCII
B15/TMIA14N10DC05T000AC05T114.notes	aIII.f46	Feb 24 1997	8201	ASCII
B15/TMIA14N11DC05T000AC05T114.notes	aIII.f47	Feb 24 1997	8155	ASCII
B15/TMIA14N12DC05T000AC05T114.notes	aIII.f48	Feb 24 1997	8145	ASCII
B15/TMIA14N13DC05T000AC05T114.notes	aIII.f49	Feb 24 1997	8206	ASCII
B15/TMIA14N14DC05T000AC05T114.notes	aIII.f50	Feb 24 1997	8194	ASCII
B15/TMIA14N15DC05T000AC05T114.notes	aIII.f51	Feb 24 1997	8192	ASCII

Waste Package Operations

Engineering Calculation Attachment

Title: CRC Depletion Calculations for TMI Unit 1

Document Identifier: B00000000-01717-0210-00007 REV 00

Attachment III, Page 2 of 21

B15/TMIA14N16DC05T000AC05T114.notes	aIII.f52	Feb 24 1997	8150	ASCII
B15/TMIA14N17DC05T000AC05T114.notes	aIII.f53	Feb 24 1997	8143	ASCII
B15/TMIA14N18DC05T000AC05T114.notes	aIII.f54	Feb 24 1997	7752	ASCII
B15/TMIA18N01DC03T000AC03T142.notes	aIII.f55	Feb 24 1997	8913	ASCII
B15/TMIA18N01DC03T142AC04T000.notes	aIII.f56	Feb 24 1997	18507	ASCII
B15/TMIA18N02DC03T000AC03T142.notes	aIII.f57	Feb 24 1997	9129	ASCII
B15/TMIA18N02DC03T142AC04T000.notes	aIII.f58	Feb 24 1997	19508	ASCII
B15/TMIA18N03DC03T000AC03T142.notes	aIII.f59	Feb 24 1997	9277	ASCII
B15/TMIA18N03DC03T142AC04T000.notes	aIII.f60	Feb 24 1997	19987	ASCII
B15/TMIA18N04DC03T000AC03T142.notes	aIII.f61	Feb 24 1997	9343	ASCII
B15/TMIA18N04DC03T142AC04T000.notes	aIII.f62	Feb 24 1997	20131	ASCII
B15/TMIA18N05DC03T000AC03T142.notes	aIII.f63	Feb 24 1997	9361	ASCII
B15/TMIA18N05DC03T142AC04T000.notes	aIII.f64	Feb 24 1997	20053	ASCII
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B15/TMIA18N07DC03T000AC03T142.notes	aIII.f67	Feb 24 1997	9327	ASCII
B15/TMIA18N07DC03T142AC04T000.notes	aIII.f68	Feb 24 1997	20055	ASCII
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B15/TMIA18N09DC03T000AC03T142.notes	aIII.f71	Feb 24 1997	9315	ASCII
B15/TMIA18N09DC03T142AC04T000.notes	aIII.f72	Feb 24 1997	19921	ASCII
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B15/TMIA18N12DC03T000AC03T142.notes	aIII.f77	Feb 24 1997	9323	ASCII
B15/TMIA18N12DC03T142AC04T000.notes	aIII.f78	Feb 24 1997	19164	ASCII
B15/TMIA18N13DC03T000AC03T142.notes	aIII.f79	Feb 24 1997	9335	ASCII
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B15/TMIA18N16DC03T000AC03T142.notes	aIII.f85	Feb 24 1997	9349	ASCII
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B15/TMIA18N17DC03T000AC03T142.notes	aIII.f87	Feb 24 1997	9299	ASCII
B15/TMIA18N17DC03T142AC04T000.notes	aIII.f88	Feb 24 1997	19790	ASCII
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B21/TMIA01N13DC05T000AC05T114.notes	aIII.f.103	Feb 24 1997	8176	ASCII
B21/TMIA01N14DC05T000AC05T114.notes	aIII.f.104	Feb 24 1997	8192	ASCII
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B21/TMIA01N16DC05T000AC05T114.notes	aIII.f.106	Feb 24 1997	8170	ASCII
B21/TMIA01N17DC05T000AC05T114.notes	aIII.f.107	Feb 24 1997	8157	ASCII
B21/TMIA01N18DC05T000AC05T114.notes	aIII.f.108	Feb 24 1997	7774	ASCII
B21/TMIA12N01DC03T000AC03T142.notes	aIII.f.109	Feb 24 1997	8880	ASCII
B21/TMIA12N01DC03T142AC04T000.notes	aIII.f.110	Feb 24 1997	18190	ASCII
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B21/TMIA12N02DC03T142AC04T000.notes	aIII.f.112	Feb 24 1997	19374	ASCII
B21/TMIA12N03DC03T000AC03T142.notes	aIII.f.113	Feb 24 1997	9113	ASCII
B21/TMIA12N03DC03T142AC04T000.notes	aIII.f.114	Feb 24 1997	19792	ASCII
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B21/TMIA12N04DC03T142AC04T000.notes	aIII.f.116	Feb 24 1997	20067	ASCII
B21/TMIA12N05DC03T000AC03T142.notes	aIII.f.117	Feb 24 1997	9289	ASCII
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Title: CRC Depletion Calculations for TMI Unit 1

Document Identifier: B00000000-01717-0210-00007 REV 00

Attachment III, Page 3 of 21

B21/TMIA12N06DC03T000AC03T142.notes	aIIIf.119	Feb 24 1997	9282	ASCII
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B21/TMIA12N16DC03T000AC03T142.notes	aIIIf.139	Feb 24 1997	9251	ASCII
B21/TMIA12N16DC03T142AC04T000.notes	aIIIf.140	Feb 24 1997	20031	ASCII
B21/TMIA12N17DC03T000AC03T142.notes	aIIIf.141	Feb 24 1997	9059	ASCII
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B21/TMIA12N18DC03T000AC03T142.notes	aIIIf.143	Feb 24 1997	8929	ASCII
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B21/TMIA26N01DC04T126AC05T000.notes	aIIIf.146	Feb 24 1997	17806	ASCII
B21/TMIA26N02DC04T000AC04T126.notes	aIIIf.147	Feb 24 1997	9828	ASCII
B21/TMIA26N02DC04T126AC05T000.notes	aIIIf.148	Feb 24 1997	19387	ASCII
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B21/TMIA26N14DC04T126AC05T000.notes	aIIIf.172	Feb 24 1997	20292	ASCII
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B21/TMIA26N16DC04T000AC04T126.notes	aIIIf.175	Feb 24 1997	9849	ASCII
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Title: CRC Depletion Calculations for TMI Unit 1

Document Identifier: B00000000-01717-0210-00007 REV 00

Attachment III, Page 4 of 21

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Title: CRC Depletion Calculations for TMI Unit 1

Document Identifier: B0000000-01717-0210-00007 REV 00

Attachment III, Page 5 of 21

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Title: CRC Depletion Calculations for TMI Unit 1

Document Identifier: B00000000-01717-0210-00007 REV 00

Attachment III, Page 6 of 21

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Title: CRC Depletion Calculations for TMI Unit 1

Document Identifier: B00000000-01717-0210-00007 REV 00

Attachment III, Page 7 of 21

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Title: CRC Depletion Calculations for TMI Unit 1

Document Identifier: B00000000-01717-0210-00007 REV 00

Attachment III, Page 8 of 21

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Title: CRC Depletion Calculations for TMI Unit 1

Document Identifier: B00000000-01717-0210-00007 REV 00

Attachment III, Page 9 of 21

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Computer

Tape Backup

File Date

File Size

File Type

Waste Package Operations

Engineering Calculation Attachment

Title: CRC Depletion Calculations for TMI Unit 1

Document Identifier: B0000000-01717-0210-00007 REV 00

Attachment III, Page 10 of 21

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Waste Package Operations

Engineering Calculation Attachment

Title: CRC Depletion Calculations for TMI Unit 1

Document Identifier: B00000000-01717-0210-00007 REV 00

Attachment III, Page 11 of 21

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Waste Package Operations

Engineering Calculation Attachment

Title: CRC Depletion Calculations for TMI Unit 1

Document Identifier: B0000000-01717-0210-00007 REV 00

Attachment III, Page 12 of 21

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Title: CRC Depletion Calculations for TMI Unit 1

Document Identifier: B00000000-01717-0210-00007 REV 00

Attachment III, Page 13 of 21

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Title: CRC Depletion Calculations for TMI Unit 1

Document Identifier: B00000000-01717-0210-00007 REV 00

Attachment III, Page 14 of 21

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Title: CRC Depletion Calculations for TMI Unit 1

Document Identifier: B00000000-01717-0210-00007 REV 00

Attachment III, Page 15 of 21

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Title: CRC Depletion Calculations for TMI Unit 1

Document Identifier: B00000000-01717-0210-00007 REV 00

Attachment III, Page 16 of 21

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Title: CRC Depletion Calculations for TMI Unit 1

Document Identifier: B00000000-01717-0210-00007 REV 00

Attachment III, Page 17 of 21

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Title: CRC Depletion Calculations for TMI Unit 1

Document Identifier: B00000000-01717-0210-00007 REV 00

Attachment III, Page 18 of 21

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D20a/TMIA20N16DC04T126AC05T000.notes	aIIIf1.168	Feb 24 1997	18844	ASCII
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Title: CRC Depletion Calculations for TMI Unit 1

Document Identifier: B00000000-01717-0210-00007 REV 00

Attachment III, Page 19 of 21

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D25a/TMIA16N06DC05T000AC05T114.notes	aIIIf1.248	Feb 24 1997	7669	ASCII
D25a/TMIA16N07DC05T000AC05T114.notes	aIIIf1.249	Feb 24 1997	7667	ASCII
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D25a/TMIA16N10DC05T000AC05T114.notes	aIIIf1.252	Feb 24 1997	7699	ASCII

Title: CRC Depletion Calculations for TMI Unit 1

Document Identifier: B00000000-01717-0210-00007 REV 00

Attachment III, Page 20 of 21

D25a/TMIA16N11DC05T000AC05T114..notes	aIIIf1.253	Feb 24 1997	7699	ASCII
D25a/TMIA16N12DC05T000AC05T114..notes	aIIIf1.254	Feb 24 1997	7653	ASCII
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D25a/TMIA16N16DC05T000AC05T114..notes	aIIIf1.258	Feb 24 1997	7616	ASCII
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D25a/TMIA25N04DC04T126AC05T000..notes	aIIIf1.264	Feb 24 1997	18594	ASCII
D25a/TMIA25N05DC04T126AC05T000..notes	aIIIf1.265	Feb 24 1997	18694	ASCII
D25a/TMIA25N06DC04T126AC05T000..notes	aIIIf1.266	Feb 24 1997	18802	ASCII
D25a/TMIA25N07DC04T126AC05T000..notes	aIIIf1.267	Feb 24 1997	18738	ASCII
D25a/TMIA25N08DC04T126AC05T000..notes	aIIIf1.268	Feb 24 1997	18718	ASCII
D25a/TMIA25N09DC04T126AC05T000..notes	aIIIf1.269	Feb 24 1997	18730	ASCII
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D25a/TMIA25N11DC04T126AC05T000..notes	aIIIf1.271	Feb 24 1997	18708	ASCII
D25a/TMIA25N12DC04T126AC05T000..notes	aIIIf1.272	Feb 24 1997	18732	ASCII
D25a/TMIA25N13DC04T126AC05T000..notes	aIIIf1.273	Feb 24 1997	18706	ASCII
D25a/TMIA25N14DC04T126AC05T000..notes	aIIIf1.274	Feb 24 1997	18748	ASCII
D25a/TMIA25N15DC04T126AC05T000..notes	aIIIf1.275	Feb 24 1997	18592	ASCII
D25a/TMIA25N16DC04T126AC05T000..notes	aIIIf1.276	Feb 24 1997	18416	ASCII
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D27/TMIA24N10DC05T000AC05T114..notes	aIIIf1.288	Feb 25 1997	7635	ASCII
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Title: CRC Depletion Calculations for TMI Unit 1

Document Identifier: B00000000-01717-0210-00007 REV 00

Attachment III, Page 21 of 21

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D28/TMIA11N18DC05T000AC05T114.notes	aIIIf1.332	Feb 25 1997	7093	ASCII
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D28/TMIA28N17DC04T126AC05T000.notes	aIIIf1.349	Feb 25 1997	17091	ASCII
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Title: CRC Depletion Calculations for TMI Unit 1

Document Identifier: B00000000-01717-0210-00007 REV 00

Attachment IV, Page 1 of 2

This attachment contains the results for the principle isotope concentrations in the depleted fuel of the Three Mile Island Unit 1 fuel assemblies. The principle isotope concentration result tables are contained on an attachment tape of this calculation file (the attachment tape has been moved to Reference 7.7). The information contained in this hard-copy representation of Attachment IV is a listing of the various files containing the principle isotope concentration result tables that are contained on the attachment tape. Each file contains the results for a given fuel assembly. The filenames identify the fuel assembly to which they correspond. The file sizes listed in the following table are the file sizes as they appear on the Hewlett Packard (HP) Series 9000 workstation. The HP file sizes differ from the file sizes on the attachment tape due to the difference in the block sizes between the HP and the personal computer. The tape containing Attachment II was written using the Colorado Model T1000e External Parallel Port Backup System for personal computers.

Filename	File Type	File Size (Bytes)	Date File Copied to Tape
B15.results	ASCII	323,814	03/18/98
B21a.results	ASCII	323,814	03/18/98
B21.results	ASCII	323,814	03/18/98
B28.results	ASCII	323,814	03/18/98
B29.results	ASCII	323,814	03/18/98
C08.results	ASCII	269,845	03/18/98
C15a.results	ASCII	269,845	03/18/98
C15.results	ASCII	269,845	03/18/98
C20.results	ASCII	269,845	03/18/98
C21.results	ASCII	269,845	03/18/98
C25.results	ASCII	269,845	03/18/98
C28.results	ASCII	269,879	03/18/98
C29.results	ASCII	269,879	03/18/98
D08.results	ASCII	161,941	03/18/98
D15.results	ASCII	161,941	03/18/98
D20a.results	ASCII	161,907	03/18/98
D20.results	ASCII	161,907	03/18/98
D21.results	ASCII	161,941	03/18/98
D25a.results	ASCII	161,907	03/18/98
D25.results	ASCII	161,907	03/18/98
D27.results	ASCII	161,907	03/18/98
D28.results	ASCII	161,941	03/18/98
E08.results	ASCII	54,003	03/18/98
E15.results	ASCII	54,003	03/18/98
E20.results	ASCII	53,969	03/18/98
E21.results	ASCII	54,037	03/18/98
E25.results	ASCII	53,969	03/18/98

Title: CRC Depletion Calculations for TMI Unit 1

Document Identifier: B00000000-01717-0210-00007 REV 00

Attachment IV, Page 2 of 2

Filename	File Type	File Size (Bytes)	Date File Copied to Tape
E27.results	ASCII	53,969	03/18/98
E28.results	ASCII	54,003	03/18/98